

# Research Operations For NSTX-U

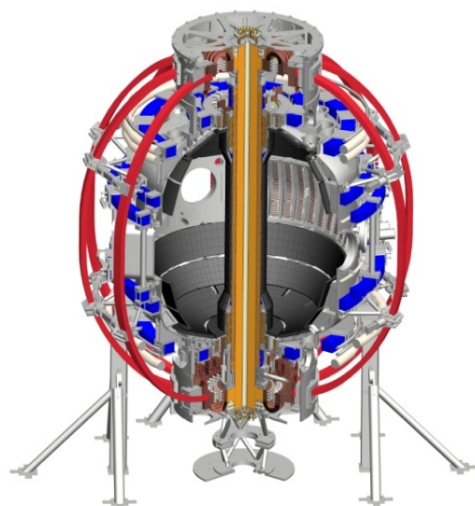
**Stefan Gerhardt**  
Research Staff

Head of Experimental Research Operations

Thanks to Brent Stratton, Bob Kaita, Masa Ono,  
Jon Menard, Charlie Gentile, & Larry Dudek

**NSTX-U Ready for Operations Review**  
**B-318**  
**December 9<sup>th</sup>, 2014**

*Coll of Wm & Mary*  
*Columbia U*  
*CompX*  
*General Atomics*  
*FIU*  
*INL*  
*Johns Hopkins U*  
*LANL*  
*LLNL*  
*Lodestar*  
*MIT*  
*Lehigh U*  
*Nova Photonics*  
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*Princeton U*  
*Purdue U*  
*SNL*  
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*U Tokyo*  
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*Inst for Nucl Res, Kiev*  
*Ioffe Inst*  
*TRINITI*  
*Chonbuk Natl U*  
*NFRI*  
*KAIST*  
*POSTECH*  
*Seoul Natl U*  
*ASIPP*  
*CIEMAT*  
*FOM Inst DIFFER*  
*ENEA, Frascati*  
*CEA, Cadarache*  
*IPP, Jülich*  
*IPP, Garching*  
*ASCR, Czech Rep*

# Outline

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- Long Term NSTX-U Research Agenda
- Recent Operations Work to Support Research Agenda
- Operations with Aquapour/CTD-425 Composite Material
- Schedule for the 1<sup>st</sup> Research Campaign
- Process for Execution of the First Research Campaign

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# Long-Term Research Agenda For NSTX-U is Defined in the 5-Year Plan

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- Available on the web at:
- <http://nstx-u.pppl.gov/five-year-plan/five-year-plan-2014-18>
- 11 Chapters, written by the entire NSTX-U team, describing
  - the research goals
  - future upgrades to the facility
- Reviewed over three days in May 2013.
- Accepted by DoE.

# Five Year Plan Described Five Highest Priority Research Goals

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## Present Upgrade

## Future Upgrade (See Backup Slides)

- 1. Demonstrate 100% non-inductive sustainment at performance that extrapolates to  $\geq 1\text{MW/m}^2$  neutron wall loading in FNSF**
  - 2<sup>nd</sup> neutral beam, higher TF
  - Cryopump (future upgrade) , NCC (future upgrade)
- 2. Access reduced  $\nu^*$  and high- $\beta$  combined with ability to vary q and rotation to dramatically extend ST physics understanding**
  - 2<sup>nd</sup> neutral beam, higher TF, higher  $I_p$
  - Cryopump (future upgrade), NCC (future upgrade)
- 3. Develop and understand non-inductive start-up and ramp-up (overdrive) to project to ST-FNSF with small/no solenoid**
  - 2<sup>nd</sup> neutral beam, higher TF
  - ECH (future upgrade)
- 4. Develop and utilize high-flux-expansion “snowflake” divertor and radiative detachment for mitigating very high heat fluxes**
  - Expanded PF-1 coil set, new divertor gas injectors
- 5. Begin to assess high-Z PFCs + liquid lithium to develop high-duty-factor integrated PMI solutions for next-steps**
  - Metal PFCs and flowing lithium systems (future upgrades)

# FY2015-16 Research Milestones Target Exploitation of New Capabilities, Exploration of New Regimes

Incremental (full ops)

Expt. Run Weeks:

FY2015

12

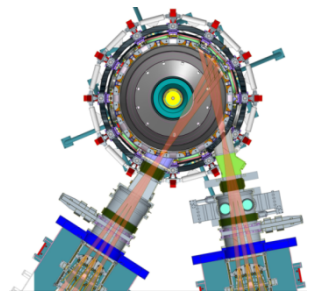
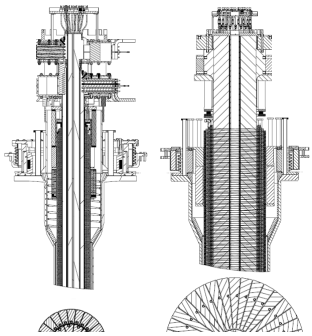
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FY2016

16

20

Previous center-stack **New center-stack**



Present NBI **New 2<sup>nd</sup> NBI**

Macroscopic Stability

Transport and Turbulence

Boundary Physics

Materials & PFCs

Waves+Energetic Particles

Solenoid-free Start-up/ramp-up

Adv. Scenarios and Control

ITER Needs + Cross-cutting

Joint Research Target (JRT)

R15-1

Assess H-mode confinement, pedestal, SOL characteristics at higher  $B_T$ ,  $I_p$ ,  $P_{NBI}$

IR15-1

R15-2

Characterize effects of NBI injection angle on fast-ion distribution and NBI-CD profile

R15-3

Develop physics+operational tools for high-performance discharges ( $\kappa$ ,  $\delta$ ,  $\beta$ , EF/RWM)

IR16-1

Assess confinement and local transport and turbulence at low  $\nu^*$  with full range of  $B_T$ ,  $I_p$ , and NBI power

R16-1

Assess heat-flux mitigation and PFC response using advanced divertor configurations + radiation at high  $q_{||-div}$

R16-2

Assess fast-wave SOL losses and core thermal / fast ion interactions at increased  $B_T$ ,  $I_p$

R16-3

Develop high-non-inductive fraction NBI H-modes for ramp-up & sustainment

Quantify impact of broadened  $J(r)$  and  $p(r)$  on tokamak confinement and stability

Assess disruption mitigation, initial tests of real-time warning and prediction techniques

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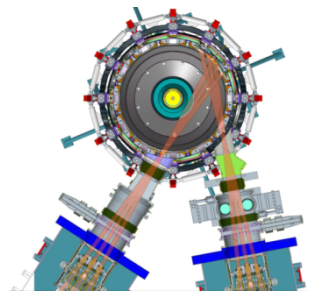
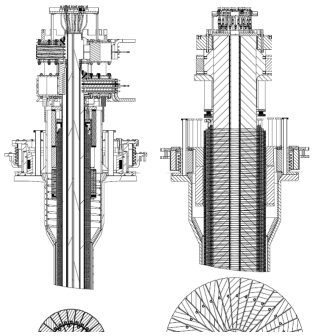
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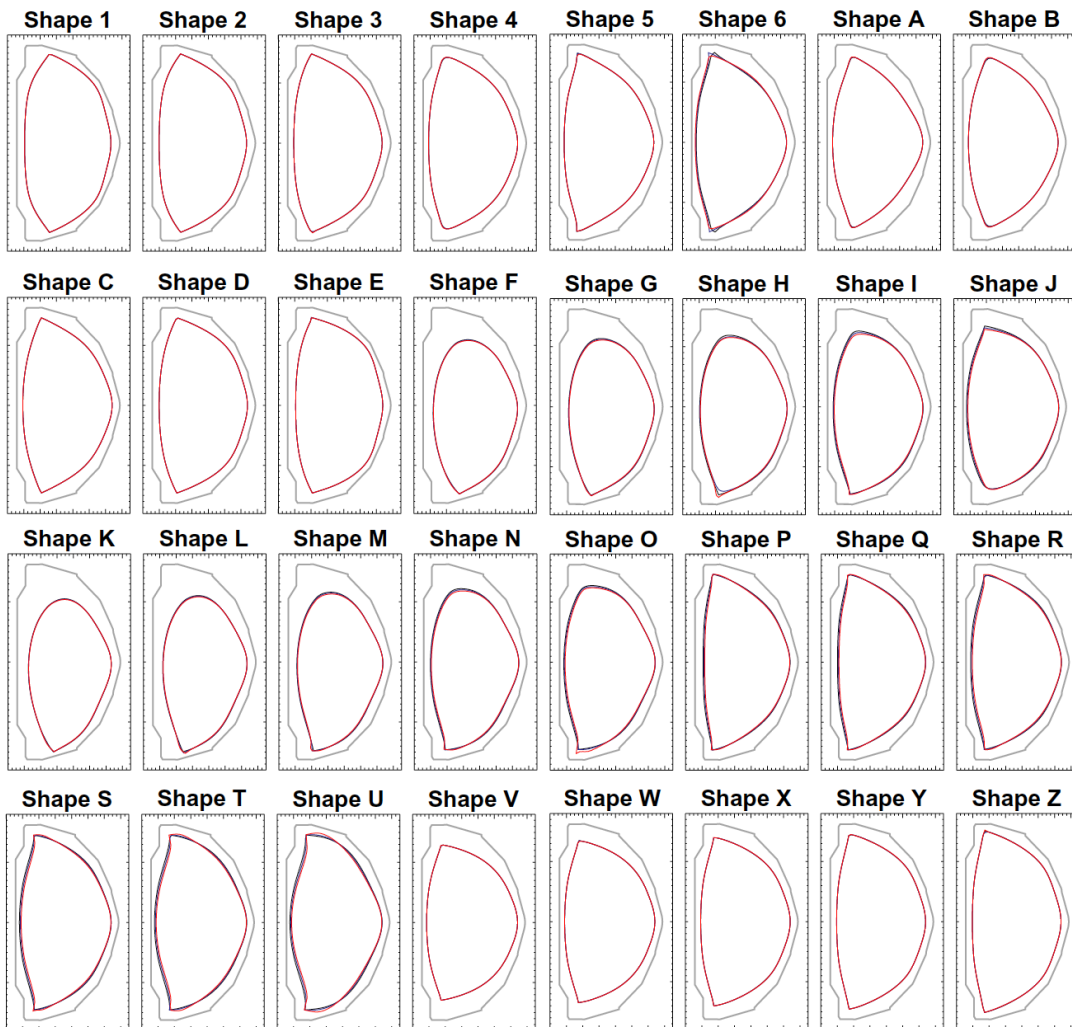
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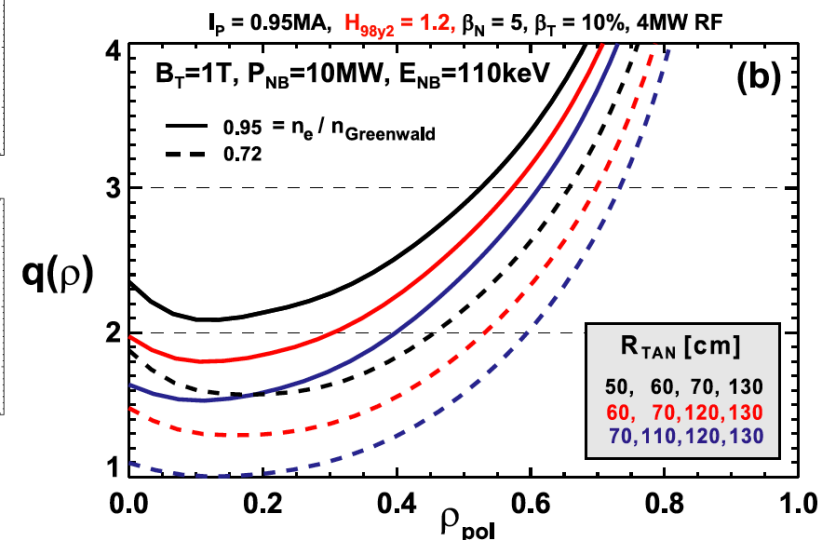
Assess disruption mitigation, initial tests of real-time warning and prediction techniques

# Engineering Design Driven By Physics Considerations



J. E. Menard, et al., Nuclear Fusion **52**, 083015 (2012)  
 S. P. Gerhardt, et al., Nuclear Fusion **52**, 083020 (2012)

- Poloidal field set consistent with a set of 96 plasma equilibria at 2 MA, 1T.
  - High and low triangularity & elongation, conventional and snowflake divertors, range of OH flux states
  - Defines CS coil locations, power supply requirements, vessel/coil forces/stresses that must be managed, some DCPS limits
- Beam tangency radii determined by integrated modeling with TRANSP.
  - Provides a range of current profiles.





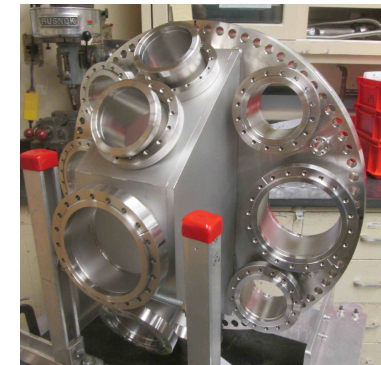
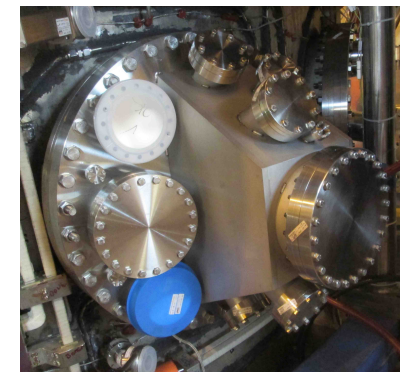
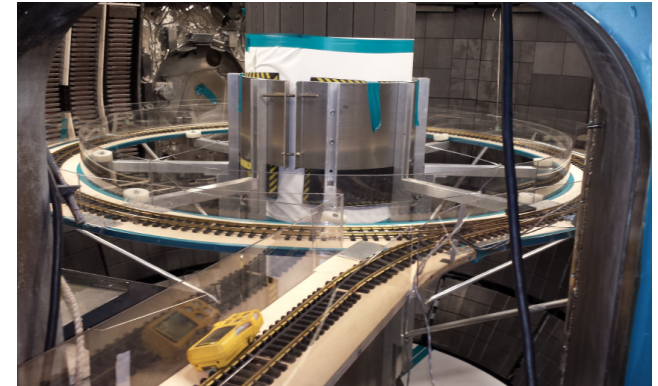
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# Research Prep. Has Been Accomplished in Concert With the Upgrade Project: **Diagnostics**

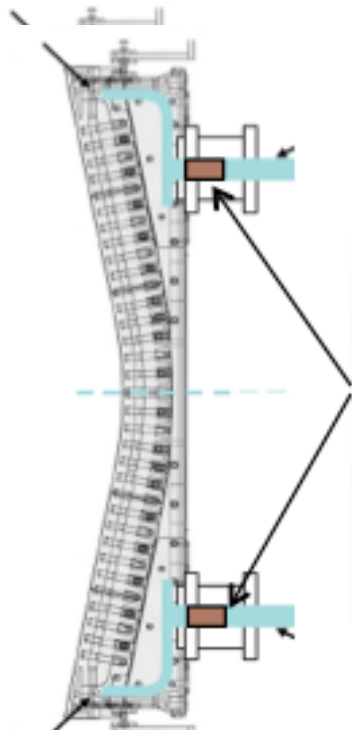
- Neutron detectors installed, and calibrated with in-vessel train track.
- Thomson scattering collection optics aligned, vacuum boundary established.
  - On track to calibrate & support experiments on the schedule to be presented later
- New diagnostics in final stage of fabrication/ installation:
  - Three new SSNPAs (UC-Irvine)
  - Bolometer, multi-energy SXR system (PPPL & JHU)
  - EUV spectroscopy systems (LLNL)
- Other major profile diagnostics reinstalled, spatially calibrated, intensity calibrated where appropriate:
  - CHERS, FIDA, T-FIDA, P-CHERS. MSE, MSE-LIF
- Magnetic diagnostics on CS expanded as part of the upgrade project.
  - And legacy magnetics on the outer vessel tested, repaired.
- Three new large port covers fabricated, installed, populated to accommodate systems displaced by vessel modifications.



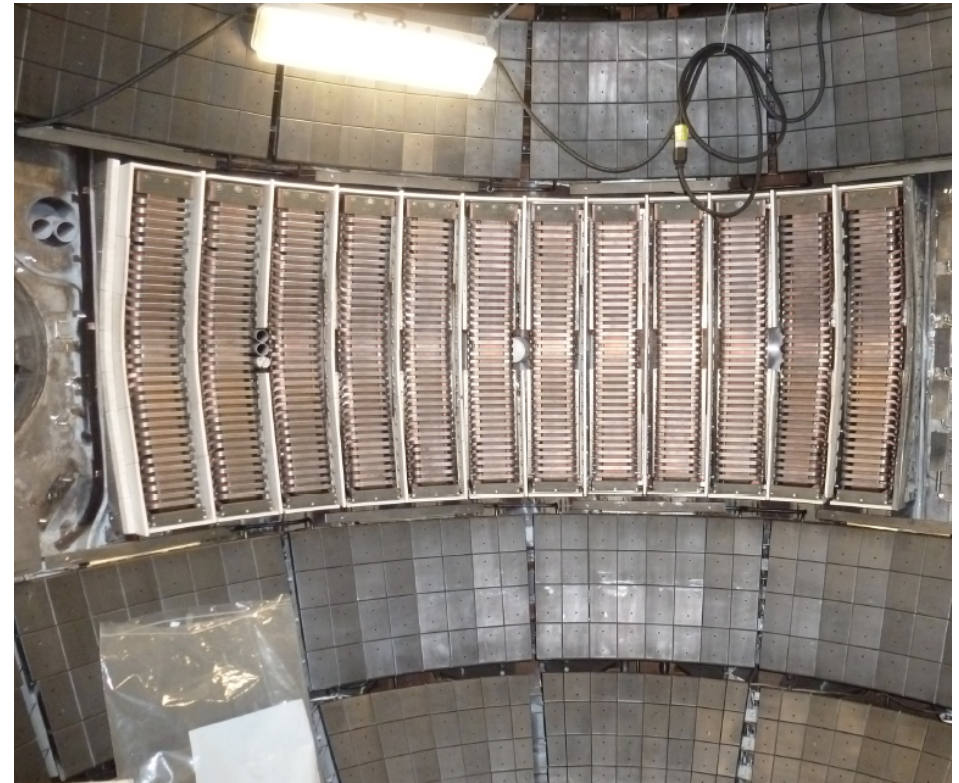
All work follows the same procedure/JHA/work package system as the Upgrade scope.

# Research Prep. Has Been Accomplished in Concert With the Upgrade Project: **HFW Operations**

**New Compliant Antenna Feeds**  
Will allow HFW antenna feedthroughs to tolerate 2 MA disruptions



Prototype electroformed compliant section



Prototype compliant feeds tested to 46 kV in the RF test-stand. Benefit of back-plate grounding for arc prevention found. Antennas were re-installed with the new feeds and back-plate grounding

- Transmission lines are in the process of being installed & tuned.
- Remaining tasks:
  - RF power supplies to be re-energized in March 2015 time-frame to be ready for research operation in May 2015.

# Research Prep. Has Been Accomplished in Concert With the Upgrade Project: **Boundary Physics Operations**

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- New boronization (dTMB) system will be used.
  - System designed through successful FDR.
  - Components have been ordered.
  - Electric service 85% complete...will use vacuum system PLC for controls.
  - Plan for it to be available for initial research operations.
- LIThium EvaporatoR (LITER)
  - LITERs were carefully stored during the outage.
  - Mounting locations are being repositioned.
  - New fume hood and other laboratory upgrades complete for LITER filling and maintenance.
  - Plan for LITER to be available for research operations.
- High-conductance divertor gas injection lines (2) have been installed
  - Supports radiative divertor studies
- Gas delivery system upgrades.
  - Moving to a uniform system of valve drive technologies.
  - All valves to be commanded from PCS.
- Materials Analysis Particles Probe (MAPP) probe has been fit-up w/ a new stand.
  - Allows material samples to be exposed to the plasma and then examined in-situ with surface science techniques.

All work follows the same procedure/JHA/work package system as the Upgrade scope.

# Research Prep. Has Been Accomplished in Concert With the Upgrade Project: **Physics Ops. And Plasma Control**

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- New firing generators for transrex recifiers designed, tested, fabricated, deployed.
- New in-house design for realtime digitizer completed, in fabrication.
- New  $I_p$  measurement systems and associate permissive generators designed, fabricated, in testing.
- Magnetics racks re-wired to support Upgrade operations, integrators now being tested.
- All low-level plasma control software has been revisited, upgraded.
  - Includes moving power supply control software from stand-alone code to an algorithm within PCS.
  - Integration of the “FCC DCPS” with associated inputs/outputs is ~85% complete.
  - System is in the final testing stages to support power supply dummy load testing.
- In the process of revisiting all physics algorithms
  - Have assigned a cognizant physicist to each algorithm to provide accountability.

All work follows the same procedure/JHA/work package system as the Upgrade scope.

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## Background: We Have Plans to Increase the Field, Current & Pulse Duration Over 3 Years

	NSTX (Max.)	FY 2015 NSTX-U Operations	FY 2016 NSTX-U Operations	FY 2017 NSTX-U Operations	Ultimate Goal
$I_p$ [MA]	1.2	~1.6	2.0	2.0	2.0
$B_T$ [T]	0.55	~0.8	1.0	1.0	1.0
Allowed TF $I^2t$ [MA <sup>2</sup> s]	7.3	80	120	160	160

1<sup>st</sup> year goal: operating points with forces up to ½ the way between NSTX and NSTX-U, ½ the design-point heating of any PF/TF coil (~75% for OH)

2<sup>nd</sup> year goal: Full field and current, but still limiting the PF/TF coil heating

3<sup>rd</sup> year goal: Full capability

**1<sup>st</sup> and 2<sup>nd</sup> year goals not affected materially by composite material**

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**This scenario most likely to be affected**

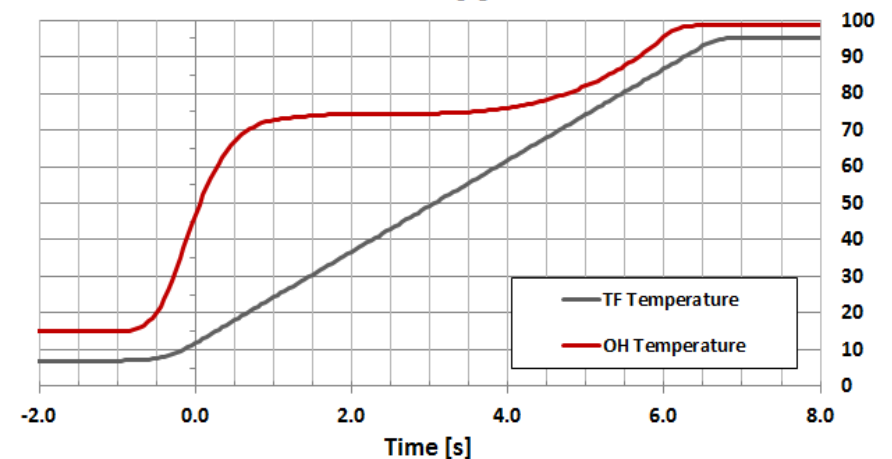
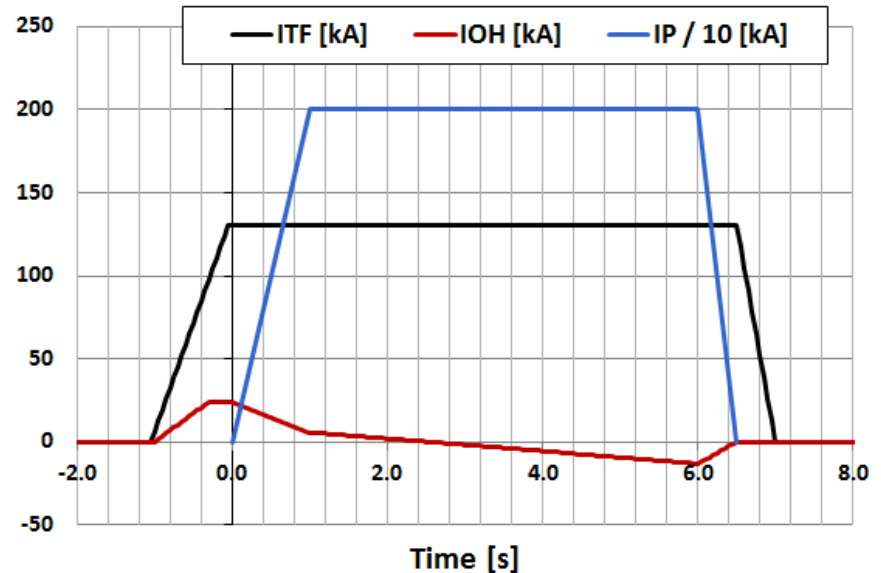
**1<sup>st</sup> and 2<sup>nd</sup> year goals not affected materially by composite material**



# Summary Statement: Physics Program Largely Unaffected By Requirement $T_{TF} < T_{OH}$

- Illustrative Example: 2 MA, 1T, 5 second.
- TF Coil:
  - Current is constant
  - Temperature is linear
- OH Coil:
  - current has a zero-crossing
  - Temperature has an “S-Shaped” curve.
- Options for maintaining  $T_{TF} < T_{OH}$ .
  - **Pre-heat the OH coil** using currents (or water) before the TF turns on.
  - Control the shape of the OH S-curve by **adjusting the amount of pre-charge**.
- In this example,
  - Full 24 kA pre-charge
  - Pre-charge duration is extended to provide heating.

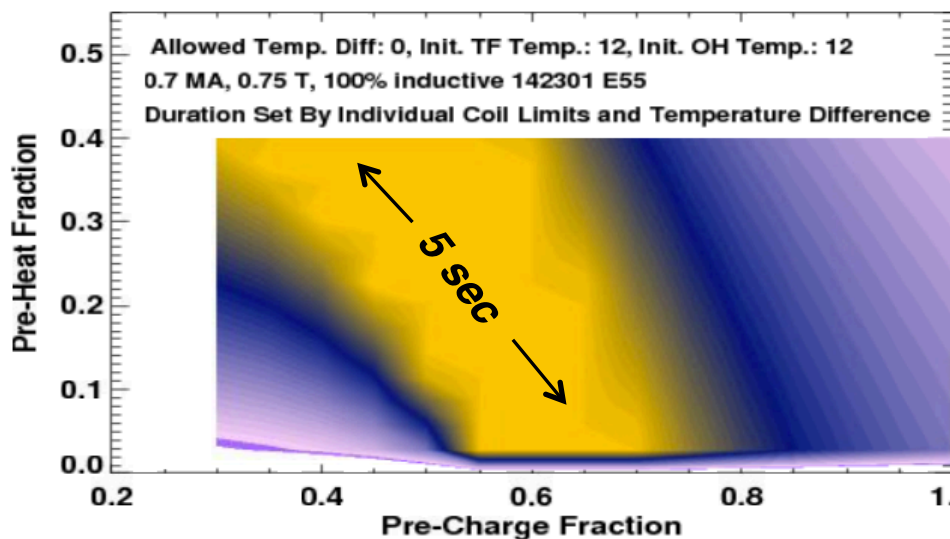
$$H_{98} = 1.2, f_{\text{Greenwald}} = 0.75, P_{\text{NBI}} = 8\text{MW}, \beta_N = 4.6$$



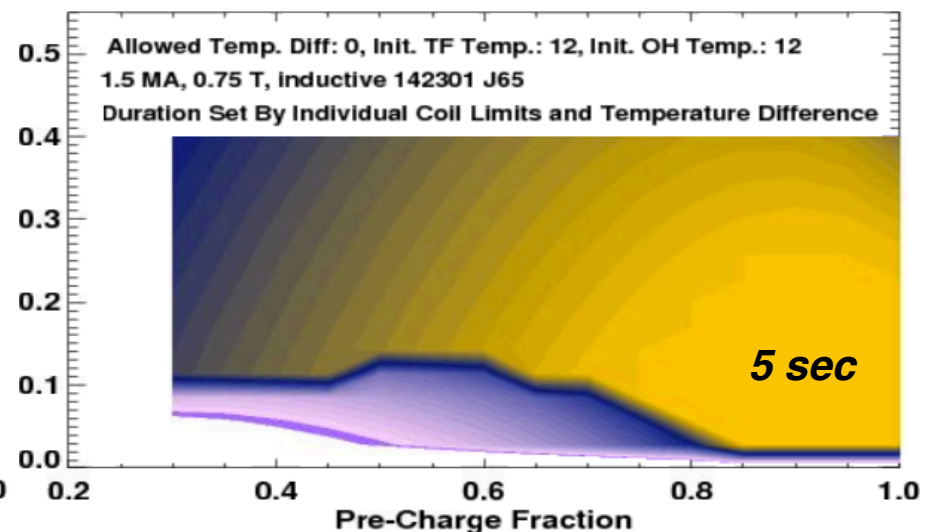
# Initial Operations Will Be Largely Unaffected By Aquapour/CTD-425 Composite

- Scan two variables in these studies:
  - Pre-heat level, quantified as the fraction of the full OH coil  $I^2t$  limit used before the shot starts.
  - Pre-Charge fraction, quantified as the fraction of 24 kA used.

**0.7 MA, 0.75 T, 100% Non-Inductive**



**1.5 MA, 0.75 T, Partial Inductive**



- Resistive pre-heat of ~15% provides operating room for 0.75 T scenarios typical of the first year.
  - Same as starting the coil at ~23 C

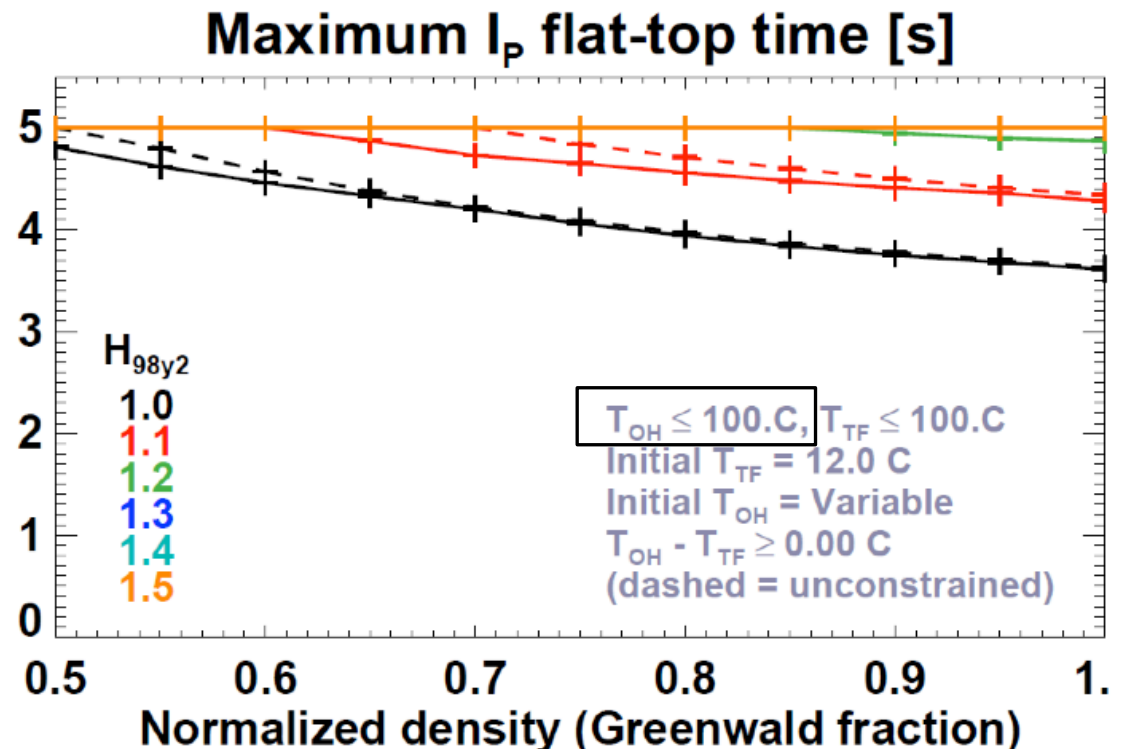
## Limitations Become Apparent, but Manageable, for 2 MA Cases

**Reason: These Often Required the Full OH I<sup>2</sup>t**

- 0D study as a function
  - Confinement ( $H_{98}$ )
  - Density (Greenwald)
- Determine via optimizer the optimum initial OH temperature, *limiting the maximum to 100 C.*
- Result:
  - Typically only a 0.2 second (or less) reduction in pulse length, provided the pre-heat is optimized
  - $H_{98} \sim 1.2$  needed for reliable 5 sec operation for any constraint on relative temperatures.

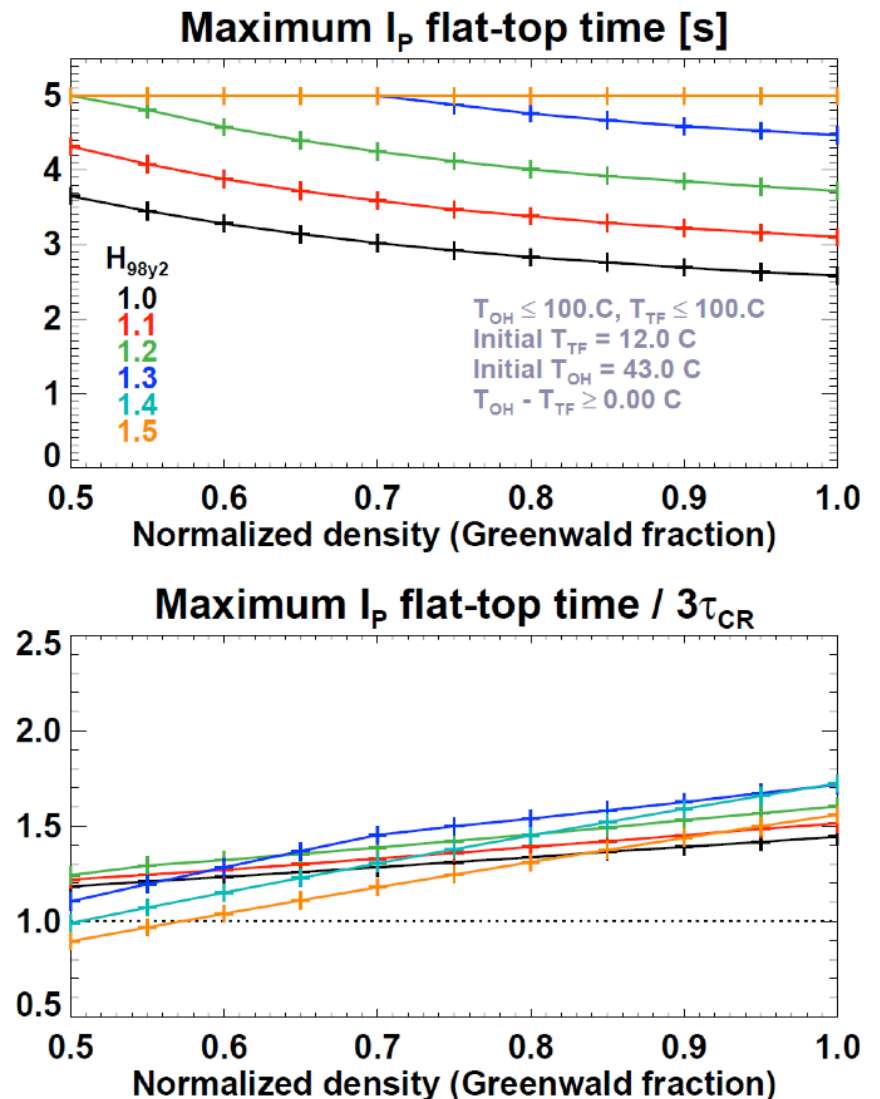
**Solid:**  
 $T_{TF} < T_{OH}$  applied as a constraint

**Dashed:**  
 No constraint on relative temperatures



## Second Optimization: Find Initial OH Temperature that Guarantees Current Profile Equilibration for a range of 2 MA Configurations

- Two assumptions in this study:
  - The important normalization for the discharge duration is the  $3\tau_{CR}$ .
  - It will be an imposition to change the initial OH temperature all the time, so need to find an optimal single value.
- Fix the initial OH temperature to 43 C
  - Could be achieved, for instance, by a “standard” OH current pulse, or the water pre-heater.
- Durations in physical units lowered for  $H_{98}=1$ , but are *greater than  $3\tau_{CR}$  for essentially all densities and confinement.*
- 110 C operation on the OH largely eliminates the composite material as an issue.
  - Optimal initial temperature drops to 36 C.
  - Plasma thermal energy confinement and CD physics, not the composite material, would have the dominant impact on the pulse duration at 2 MA.
  - Under engineering evaluation, looking quite feasible.



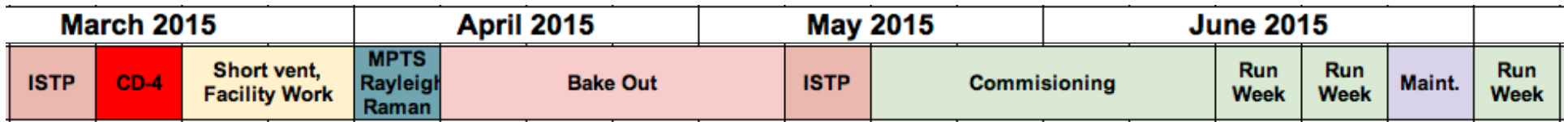
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# Activities between CD4 and Research Run

Important to optimize them to start the timely research run



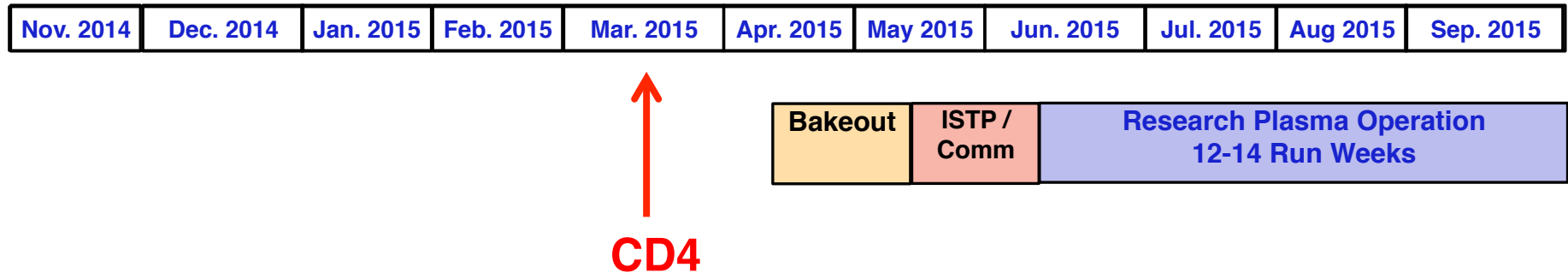
Short Vent to Install New Diagnostics (no manned entry!) **only if needed.**  
 Data Acquisition & Diagnostics Commissioning  
 MPTS Rayleigh/Raman Calibrations  
 Ex-Vessel Diagnostic Installations  
 Gas Delivery System Final Checks  
 Magnetics Debugging  
 Beamline #1 Final Checks  
 HHFW Tube Commissioning

Calibrations:  
 Magnetic  
 MSE  
 CHERS  
 Neutrons  
 Boronization / GDC

Commissioning  
 $I_p \leq 1.0$  MA,  $B_T \leq 0.5$   
 Breakdown and current ramp scenarios  
 EFIT & rtEFIT Reconstructions  
 Shape & position control  
 Reliable H-mode  
 Diagnostic operations  
 DCPS under plasma operations  
*Boronization +He GDC for PFCs*

If some tasks can be moved before the CD-4 date without impacting that schedule, we will do so.

# Nominal NSTX-U run schedule for FY2015



- ~ 2 month period allocated between CD-4 and plasma operations → CD-4 in mid-March, plasma ops in mid-May
- Plan: 12-14 run weeks (assumes ~1 maintenance week / month)
- If machine is running well at end of FY15, may run into early FY16
  - Provide additional data for APS 2015 and IAEA synopses for 2016

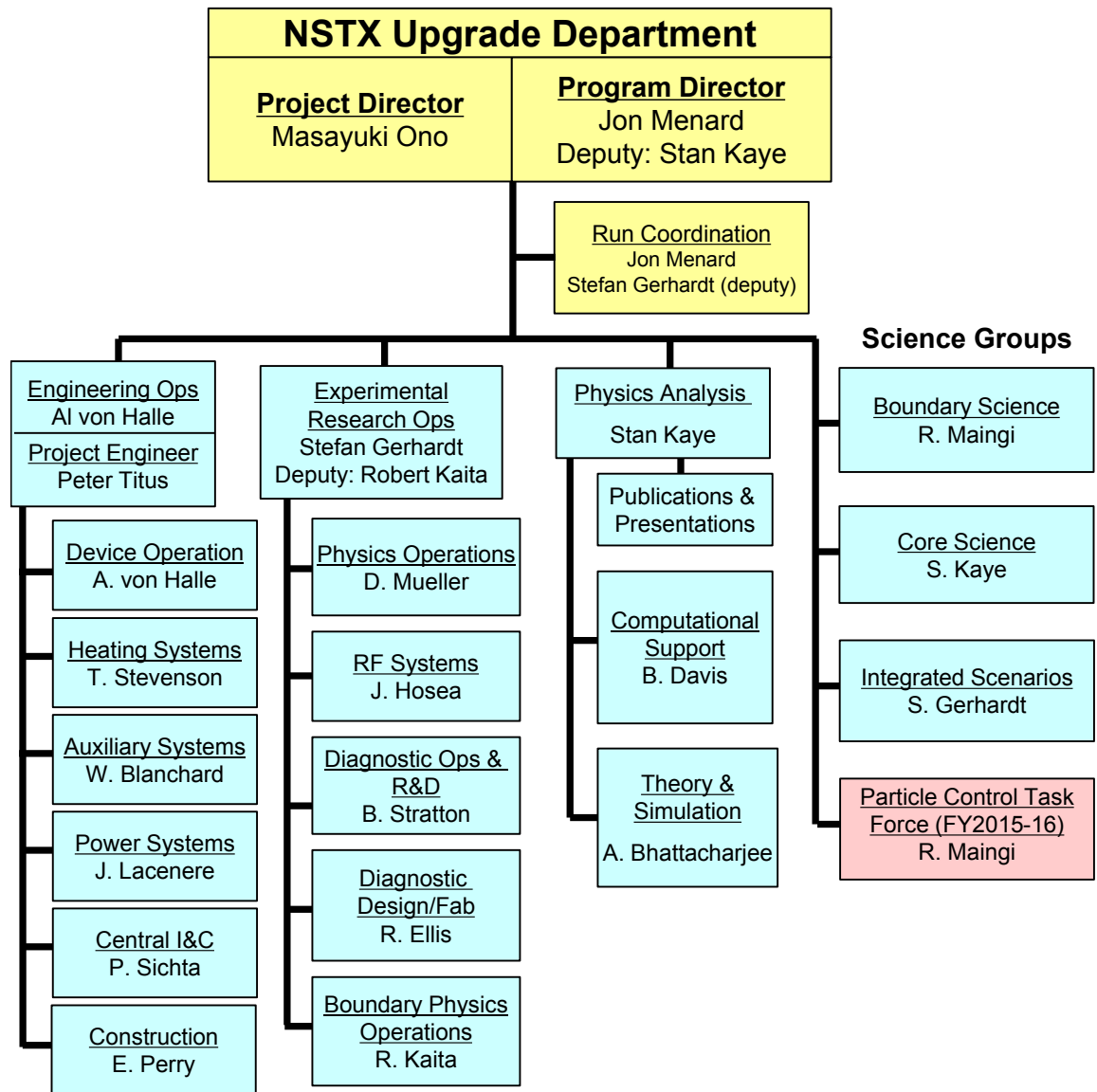
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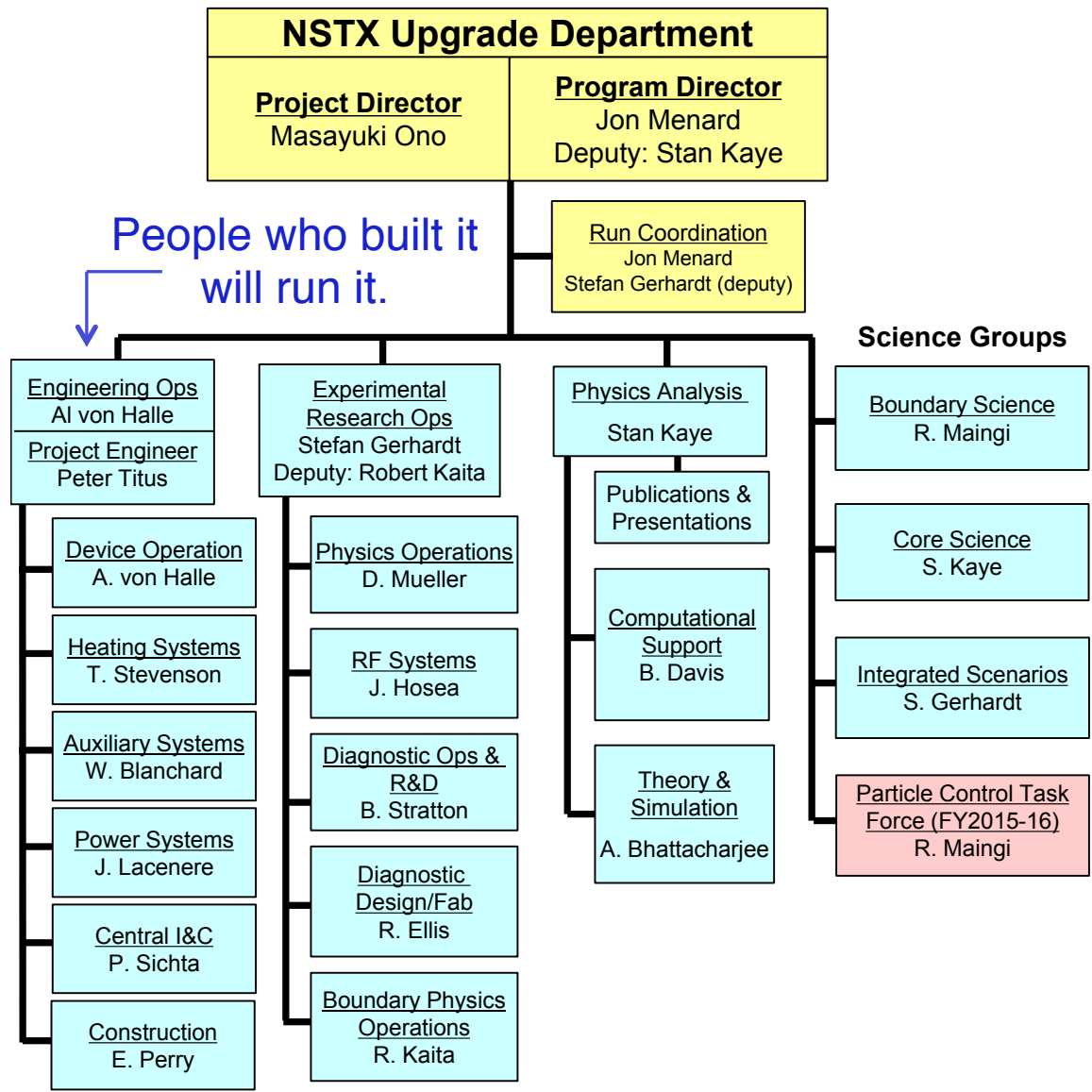
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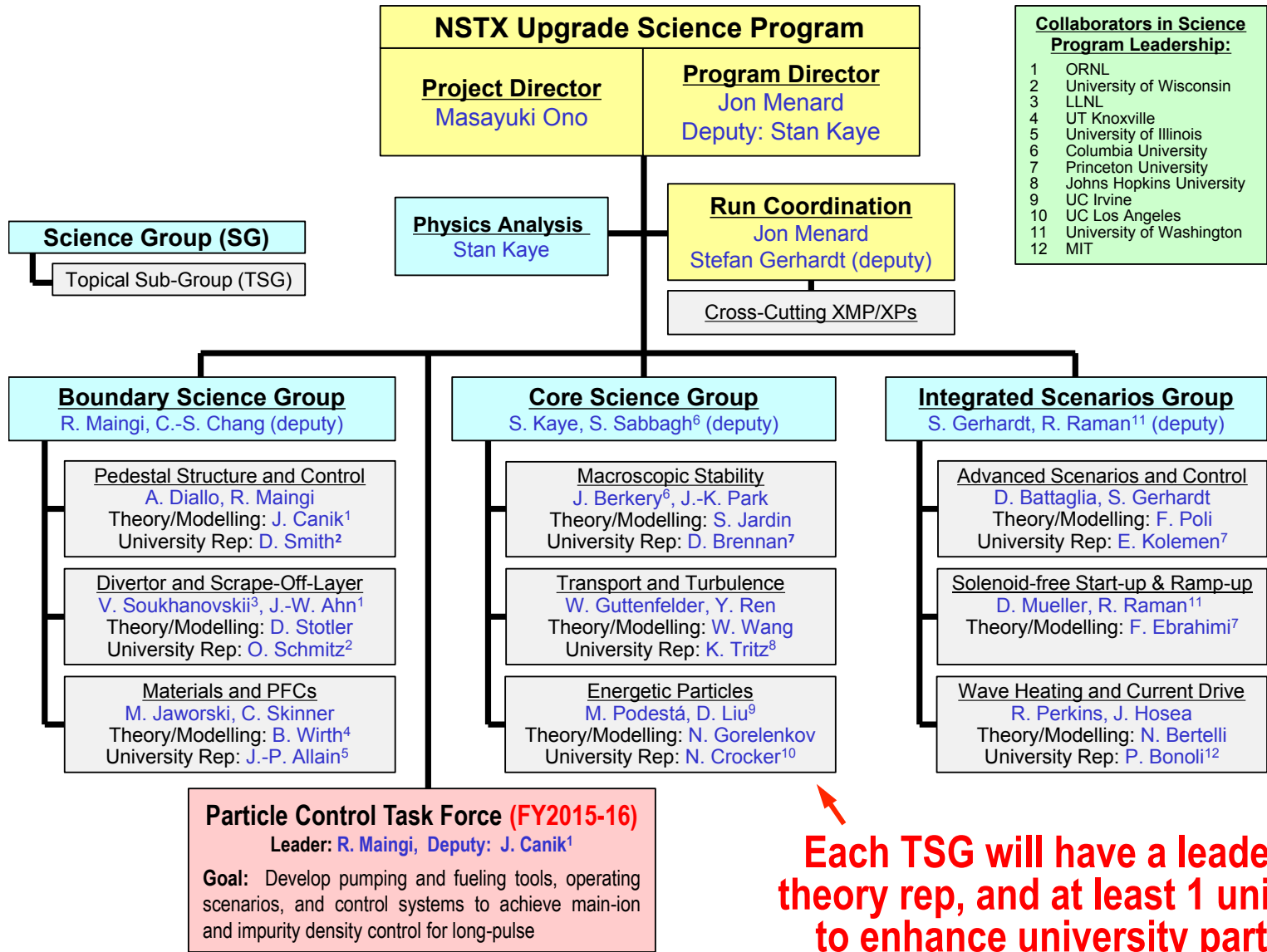


People who built it will run it.

Work at D-Site is governed by the WCC & OP-AD-56, which defines the chain of command and various rolls (COE, D-Site supervisor, Health Physics,...).

Experimental program management goes through Engineering Ops. division to get work done.

# NSTX-U Research Program Will be (re-)Organized Along 3 “Science Groups” and 9 TSGs for the FY15 run



## Topical Sub Groups (TSGs) play major role in governing the research program

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- Led brainstorming, organization, writing of 5 year plan
- Address highest priority scientific issues through discussion and consensus at open meetings
- Organize the NSTX-U Research Forum sessions.
- Draft scientific milestones
- Propose and execute experiments
- Define facility and theory resources to achieve research goals
- Aid dissemination of results with Physics Analysis Division
  - Journal publications, invited talks, seminars, colloquia, conferences, ITPA, BPO
- Provide summaries of scientific progress at NSTX-U team meetings and other venues to promote discussion
- Assist and report to the NSTX-U Program and Project directors

# Details of the Research Program Will Be Defined By The Team in a Series of Meetings

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- Research Prep. Meeting: Jan. 28<sup>th</sup> and 29<sup>th</sup>
  - Designed to notify the scientific team of facility status
  - Day #1 talks:
    - Gerhardt : Intro, Magnetics
    - Von Halle : Engineering Operations (Power systems, NBI, this review,...)
    - D. Mueller : Physics Operations (operator training, PCS,...)
    - Kaita : Boundary Operations (GDC, LITER, Boronization,...)
    - Stratton : Diagnostic Operations
    - Hosea : RF Operations
  - Day #2: Update from SG/TSGs on research goals and XMP/XP solicitations
- Experimental proposals (XPs) submitted by through a dedicated web site.
  - Historically 1.5-2x more run times requested than is available.
- Research Forum (Feb 24<sup>th</sup> -27<sup>th</sup>, PPPL)
  - Opening plenary session summarizes anticipated run schedule & programmatic goals.
    - Include run time allocations, typically 1/3 of the available run time held in reserve.
  - Breakout sessions for each TSG allow open discussion of research proposals and preliminary XP prioritization.
  - Summary plenary session presents the preliminary prioritization.

# What Exactly is An XP?

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- What is an XP?
  - It is an official PPPL document, archived in the ops. center.
    - Process defined in OP-ADX-03
  - Subject both optional TSG and mandatory team reviews.
  - Author is the scientist who leads the XP.
  - Signed by SG or TSG leader
  - Run coordinator signature takes the place of the RLM.
  - Describes both required machine configuration and individual steps of the experiments.
  - Typically describes 1-2 days of run time.
  - Successful completion of an single XP should lead to a publication or invited talk.
- We also have XMPs.
  - ~2-4 hour blocks of run time designed to test a specific piece or hardware or control software.
  - Process defined in OP-ADX-02
- Weekly “program/ops” meeting where the run coordinator provides a draft ~2-3 week schedule, which is then iterated with the research and engineering team before posting.

# Physics Operations Staff+Collaborators Will Be Ready to Execute the NSTX-U Research Program

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- Three NSTX physics operators will return to NSTX-U.
  - D. Mueller is a world-recognized tokamak driver.
    - Operated TFTR
    - Has collaborated on EAST and K-STAR control development over the last year.
    - Author of the CD-4 XP.
    - Author of the machine commissioning XP.
  - D. Battaglia has spent the last 2 years as a DIII-D operator
  - R. Raman (U. of Washington) provides leadership in CHI, MGI areas + physics operations.
- D. Mueller will hold a training/refresher course following CD-4.
  - Plan to train an additional 2-3 operators.
  - Slides from previous course:
    - [http://nstx.pppl.gov/DragNDrop/Operations/Physics\\_Operations\\_Course/](http://nstx.pppl.gov/DragNDrop/Operations/Physics_Operations_Course/)
- Major diagnostics have primary and backup support.
  - Magnetics: S. Gerhardt + new post-doc.
  - CHERS: R. Bell + M. Podesta
  - Thomson Scattering: B. LeBlanc+A. Diallo+new post.-doc.

## Summary

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- NSTX-Upgrade project directly feeds the long term goals for the facility.
- Operations work, in concert with the Upgrade scope, has progressed well.
- Operations scenarios compatible with the Aquapour/CTD-425 composite have been identified.
- The process for defining the research run is in place.

*The NSTX-U team is excited and prepared to move to research operations!*



# Backup

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# Some Configuration Information

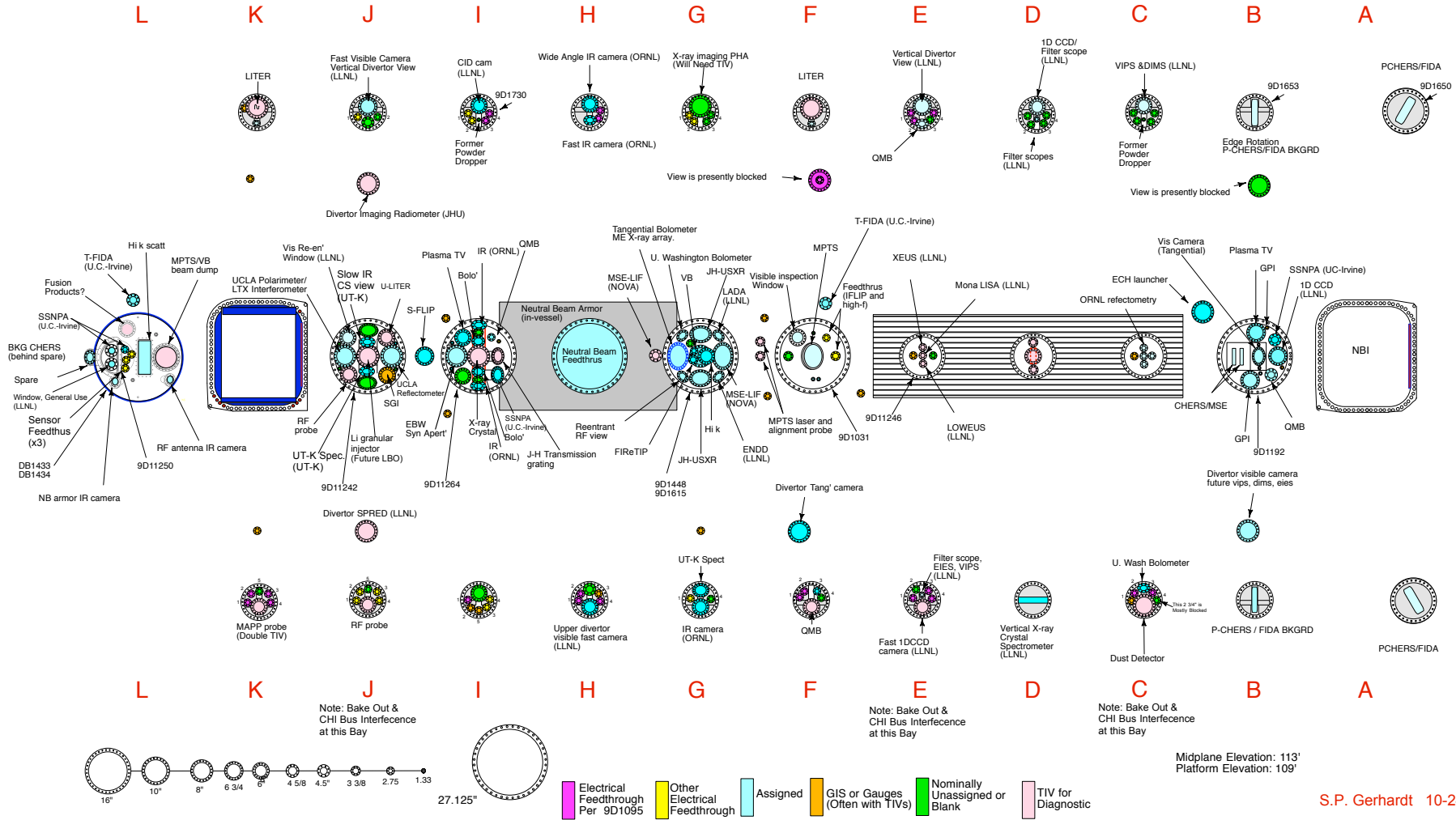
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# NSTX-U Ports are Assigned to Diagnostics

NSTX Cat. 4 Tile Diagnostics: NSTX345  
 NSTX Cat. 3 Tile Diagnostics: ED1324  
 NSTX Vessel Mounted Diagnostics: 9D11266  
 NSTX RWM Coil Details: DC1329

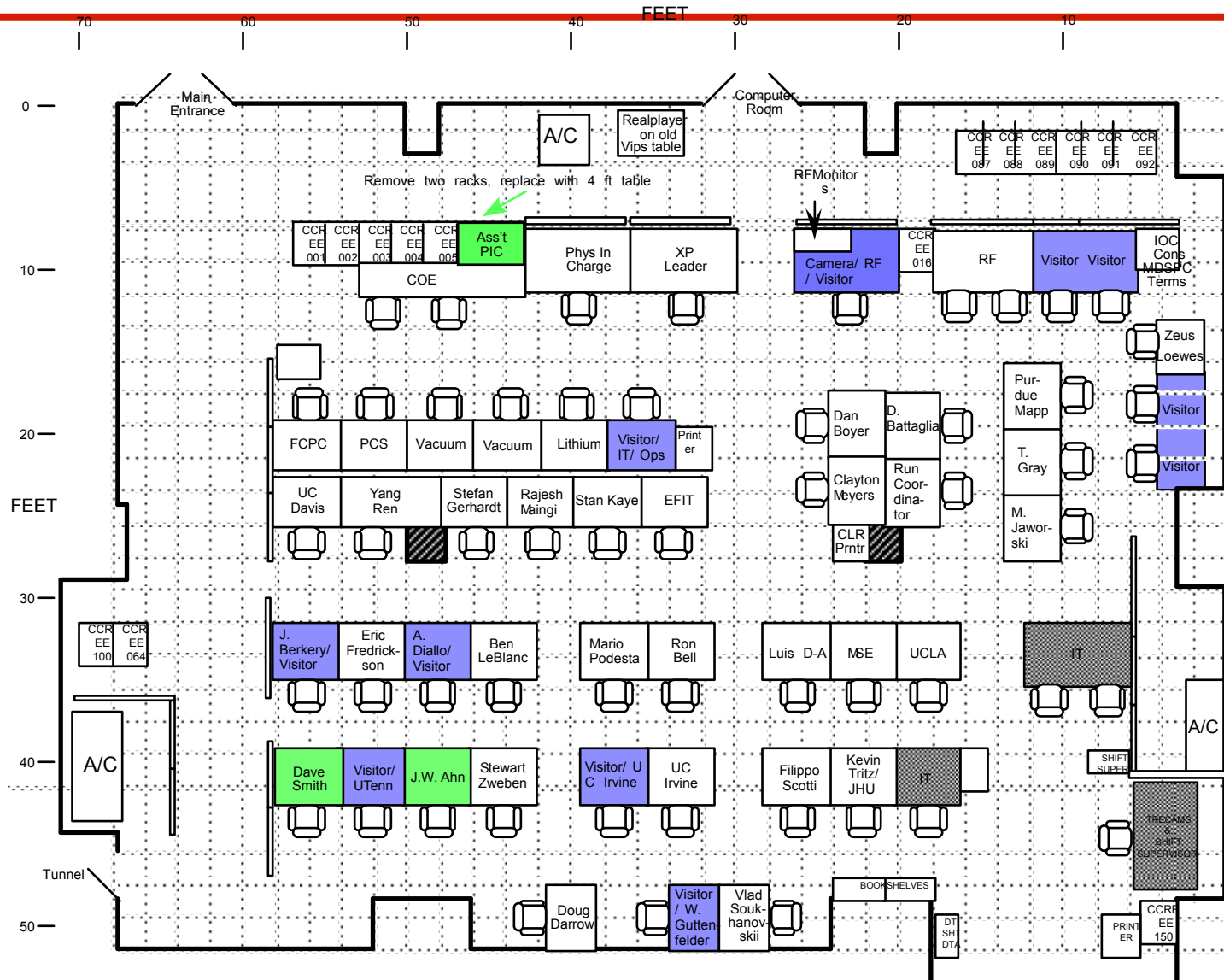
## Port assignment for 2015 Operations Gas Injectors and Ion Gauges on Separate Port Drawing

13 1/4" Port Cover Drawings  
 9D11270  
 9D1730  
 9D1917  
 9D1598



S.P. Gerhardt 10-24-2014

# Seat Assignments in the Control Room Have Been Updated



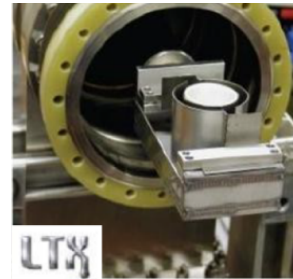
# Long Term Upgrades

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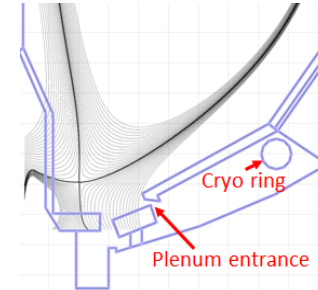
# NSTX-U facility enhancements proposed for 5 year plan support FESAC Tiers/Priorities

- Improved particle control tools
  - Control D inventory, rapidly trigger ELMs to expel impurities (*Transients, PMI*)
  - Low  $\nu^*$  to understand ST confinement to support FNSF, validation (*FNSF, Predictive*)
- Disruption avoidance, mitigation (*Transients, Predictive*)
  - Massive gas injection, detect halos, disruptions, control  $\nu_\phi$ , RWM, ELM
- ST start-up and ramp-up tools (*FNSF*)
  - ECH to raise start-up plasma  $T_e$  to enable FW + NBI + BS  $I_p$  ramp-up
  - Test EBW-CD start-up, sustainment
  - Start-up/ramp-up critical for ST-FNSF
- Begin transition to high-Z PFCs, assess flowing liquid metals (*PMI, FNSF*)
  - Plus divertor Thomson, spectroscopy

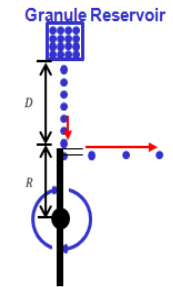
Upward Li evaporator



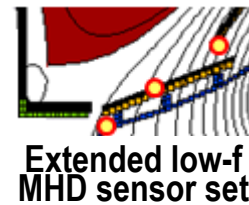
Divertor cryo-pump



Li granule injector (LGI)

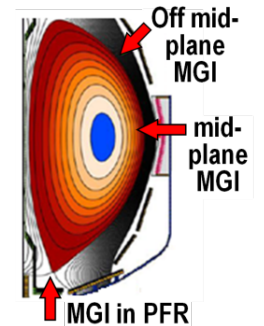
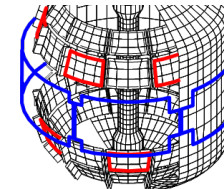


Disruption avoidance, mitigation (*Transients, Predictive*)



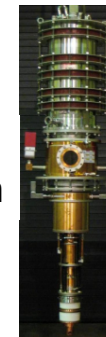
Extended low-f MHD sensor set

Midplane + off-midplane non-axisymmetric control coils (NCC)



- ST start-up and ramp-up tools (*FNSF*)
  - ECH to raise start-up plasma  $T_e$  to enable FW + NBI + BS  $I_p$  ramp-up
  - Test EBW-CD start-up, sustainment
  - Start-up/ramp-up critical for ST-FNSF

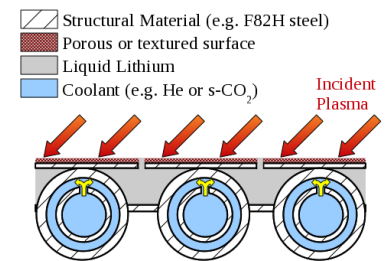
1MW 28 GHz gyrotron



High-Z tiles



Actively-supplied, capillary-restrained, gas-cooled LM-PFC



## 4 Major Upgrade Are Being Considered for the next 5 Years

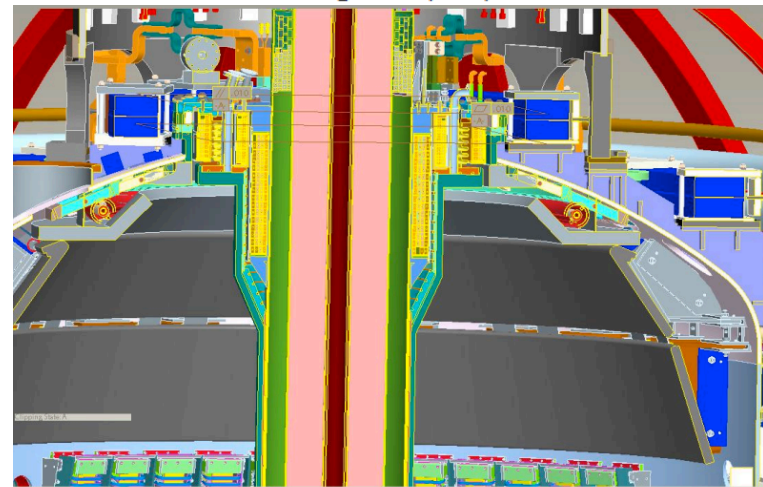
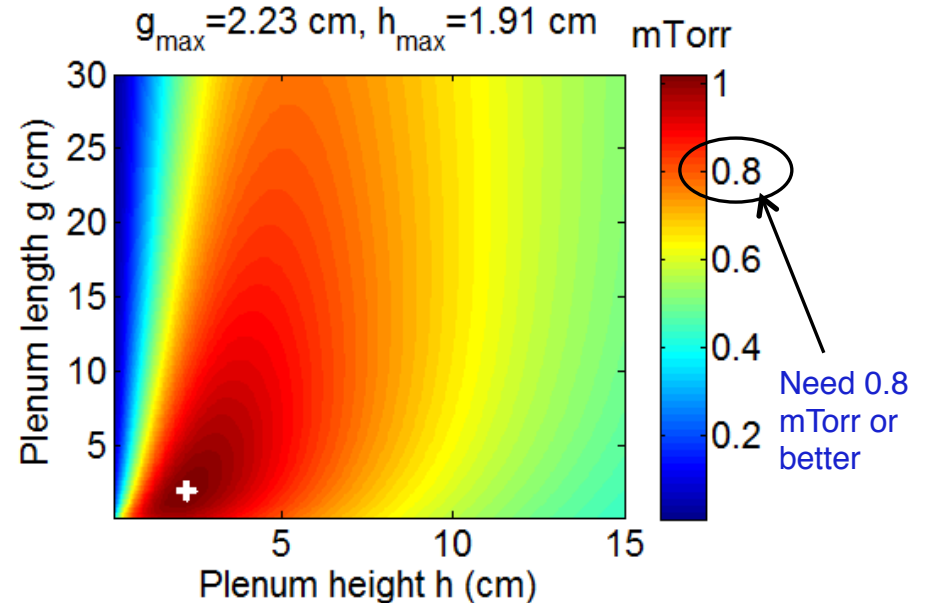
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- We need robust density control.
  - Proposed Upgrade: *Divertor cryopump*
- We need to develop high-performance ST scenarios compatible with reactor relevant plasma facing components (PFCs).
  - Proposed Upgrade: *High-Z PFC upgrade*
- We need to understand how a CHI formed plasma can be ramped to full current without solenoid induction.
  - Proposed Upgrade: *ECH system*
- We need to better understand and optimize error field correction, resonant magnetic perturbations for pedestal control, fast feedback for RWM control, rotation profile control.
  - Proposed Upgrade: *Additional 3D field coils*

For detailed justification, see: <http://nstx-u.pppl.gov/five-year-plan/five-year-plan-2014-18>

# Cryopump Physics Design to Provide Pumping over a Wide Range of Divertor Geometries and Core Densities

- Physics design completed in collaboration with ORNL<sup>1</sup>.
  - Defined the geometry, plenum sizes, ability to pump various geometries.
- Conceptual design process has been initiated:
  - Draft GRD has been formulated.
  - Initial designer sketches of in-vessel implementation completed.
  - Potential refrigerator systems and associated elements identified.
  - Goal is to to have the system available for the 2017 run campaign.

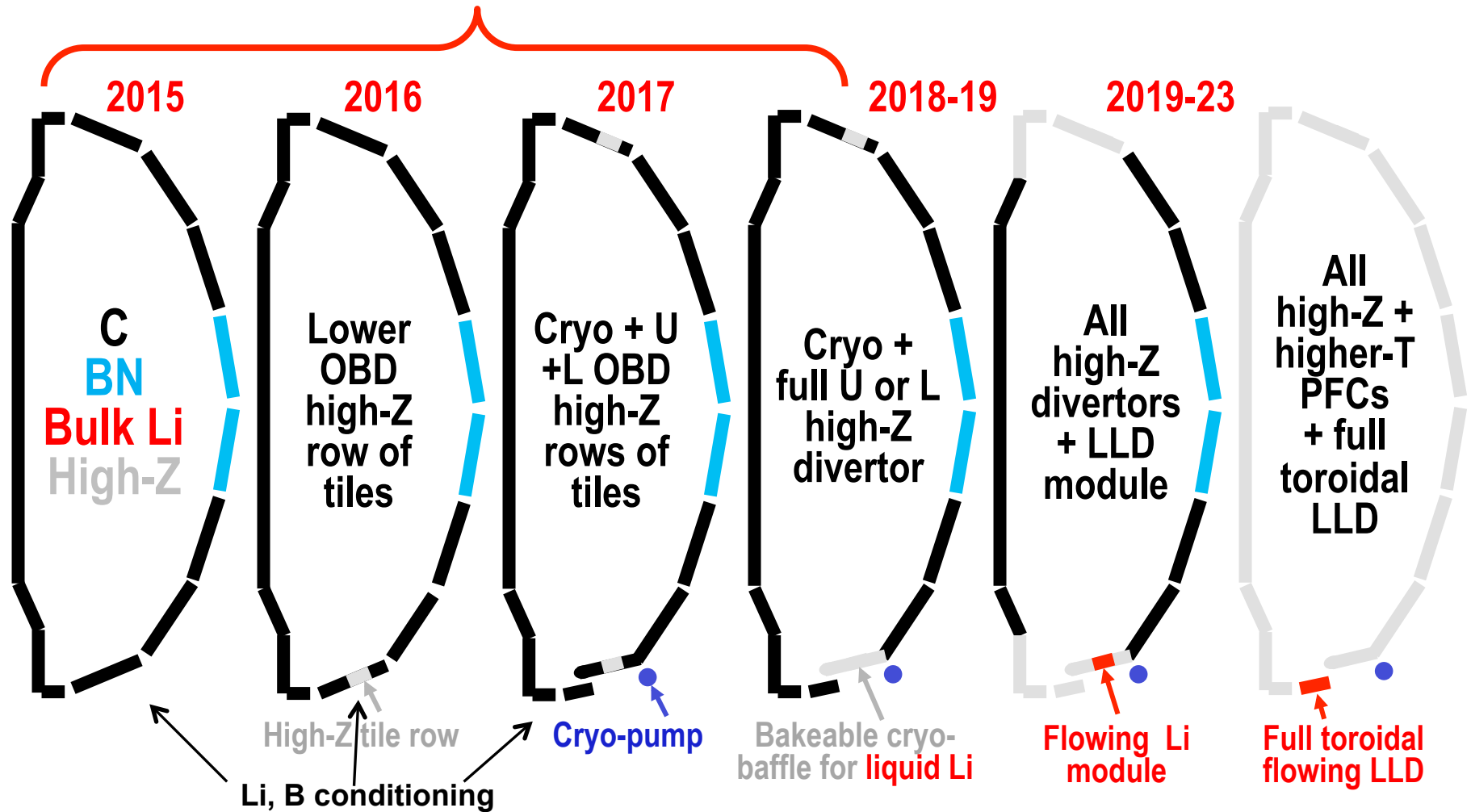


[1] [http://nstx.pppl.gov/DragNDrop/Scientific\\_Conferences/APS/APS-DPP\\_12/Contributed\\_Posters/PP8.00030\\_Canik\\_APS2012.pdf](http://nstx.pppl.gov/DragNDrop/Scientific_Conferences/APS/APS-DPP_12/Contributed_Posters/PP8.00030_Canik_APS2012.pdf)



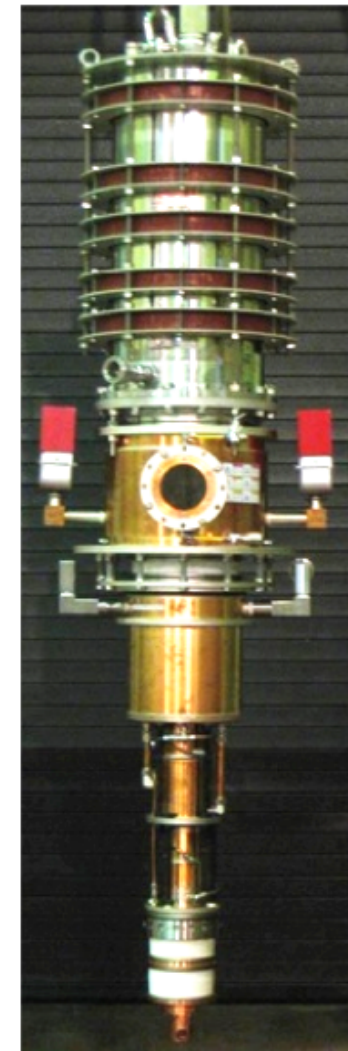
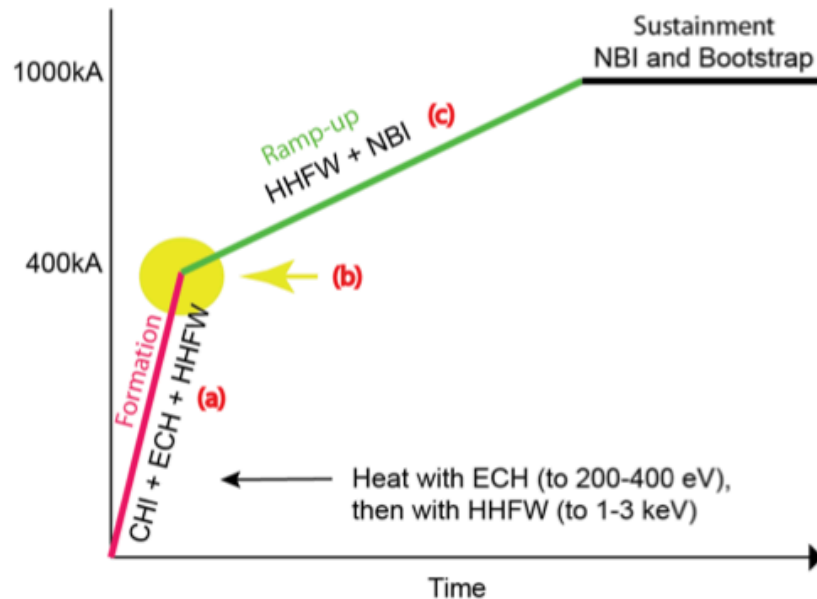
# Stages Plan Has Been Developed to Implement High-Z PFCs & Flowing Liquid Metal Systems

*Nominal 2014-18 5 year plan steps for implementation of cryo-pump + high-Z PFCs + LLD*



# 28 GHz Gyrotron System Will Facilitate Non-Inductive Ramp-Up

## NSTX-U Start-up and Ramp-up strategy



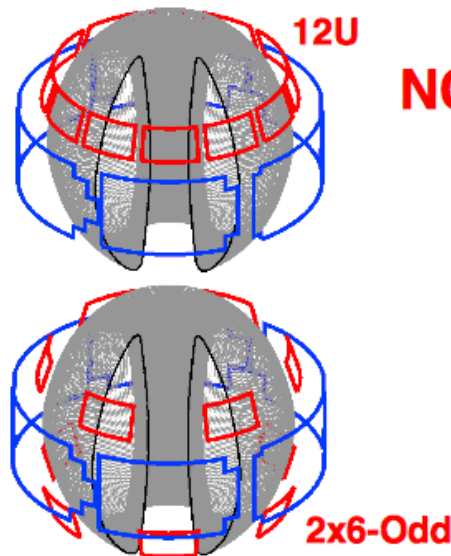
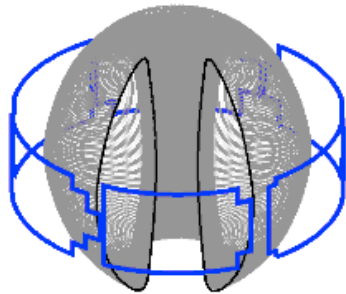
- Coaxial Helicity Injection can form a 200-400 kA seed plasma, but it is too cold and quickly decays.
- Use of ECH can “bridge the gap”, to where HHFW and then NB current drive can support the ramp and sustain the current.
  - 20-40% first pass absorption predicted by GENRAY.
- Goal of first ECH power in 2017 run

1 MW Tube Developed for Gamma 10

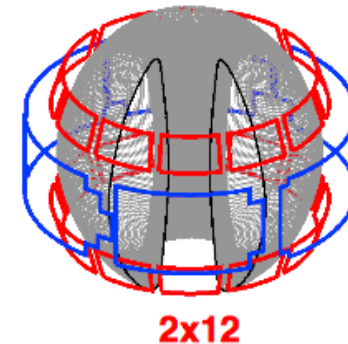
# 3D Coil Physics Design Targets a Range of Physics Objectives

- NCC = Non-axisymmetric Control Coil
- Evaluated three upgrade options based on numerous physics criterion
  - Magnetic breaking, error field control, fast RWM control, RMP applications.
- Initial findings shows that while the 2x12 options is best, a phase implementation starting with the 2-6-Odd approach may be a good intermediate step.

**Existing  
Midplane coils**



**NCC Options**



**2x12**

*Targeting the Out  
Years of the 5 Year Period*

# Collaborators Play a Key Role in the NSTX-U Research Program

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- University, national lab, and business collaborations for both data analysis and facility upgrades (diagnostics, gas injectors,...).
  - Are X of X topical science group leaders, 2 of three science group deputies.
- Collaborations reviewed & renewed on a 3-4 year cycle.
- Key documents:
  - Record of Discussion: documents communications between PPPL and collaborator during the formulation of DOE proposal, including estimates of PPPL resources to support collaboration is funded
  - Record of Agreement: agreed commitments of resources, equipment, and facilities by collaborator and PPPL.
  - Data Usage Agreement: access to and publication of data.
- PPPL generally provides the vacuum interface, floor space, AC power & other services for diagnostics.
  - Collaborator provides the diagnostic itself, typically including data acquisition.
- Collaborators have the same safety & training requirements as PPPL employees.
  - And their systems have the same design reviews and work package requirements.

# Analysis Justification for $T_{TF} < T_{OH}$ Rule

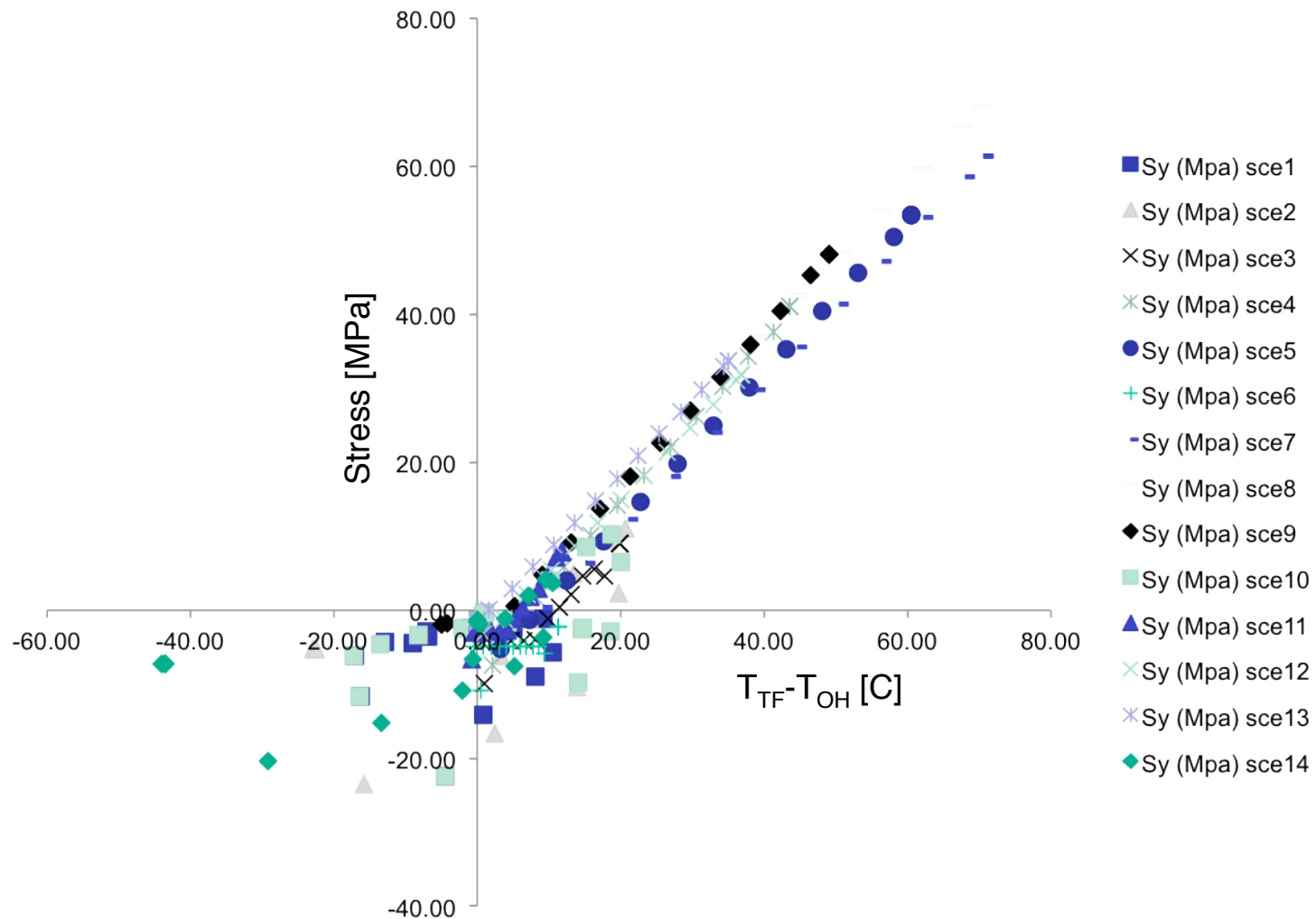
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# Analysis Supports the Use of Temperature Differentials for The Initial Protection Scheme: Method

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- Created 14 different discharge scenarios.
  - Mostly 2 MA, 1T, but a few at lower field and current.
  - Many variations in the pre-charge and pre-heat.
  - All had the TF temperature eventually exceed the OH temperature, sometimes by a large amount.
    - So are useful for defining protection scheme.
  - Had a wide range of OH states during the time when  $T_{TF}$  exceeded  $T_{OH}$  by 0-10 C.
- Used ANSYS to analyze the OH stress at 18 times in each of the discharge scenarios.
  - 14x18=252 combinations of stress, temperature difference, OH state
- Motivation: Find a bounding curve for the OH stress that is a function of only the temperature difference.

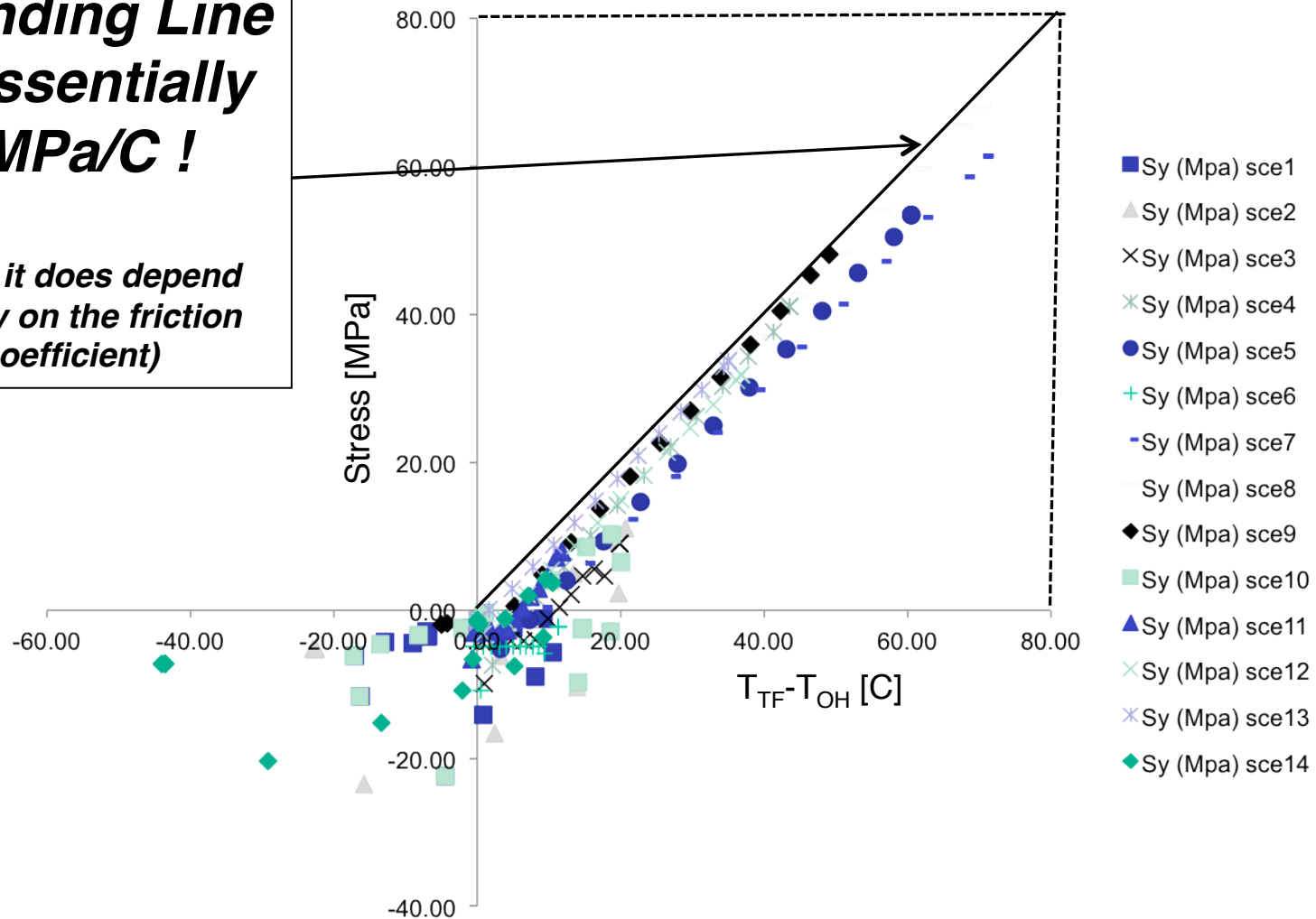
# Analysis Supports the Use of Temperature Differentials for The Initial Protection Scheme: Result (I)



# Analysis Supports the Use of Temperature Differentials for The Initial Protection Scheme: Result (I)

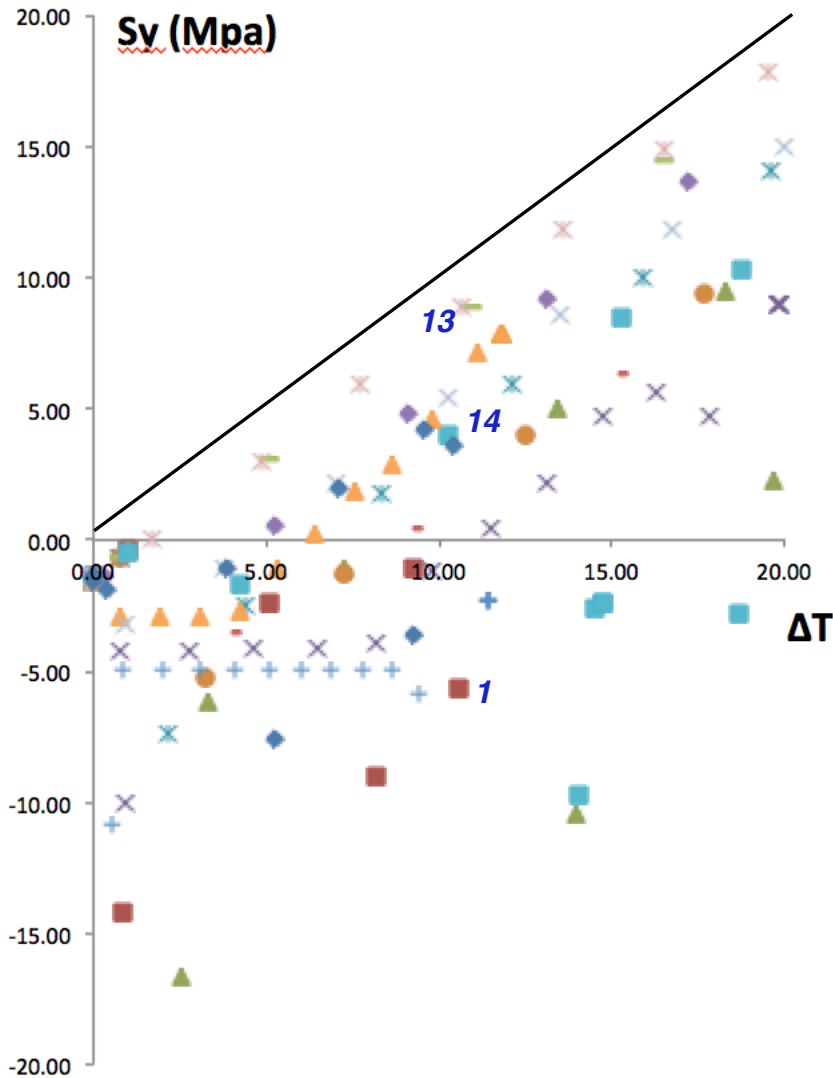
**Bounding Line  
is Essentially  
1MPa/C !**

*(Note, it does depend  
weakly on the friction  
coefficient)*





# Analysis Supports the Use of Temperature Differentials for The Initial Protection Scheme: Result (II)



- Unity-slope bounding line holds over the temperature different region of interest ( $0 < \Delta T < \sim 20$ ).
- Large variation under that line, due to:
  - The OH state
  - Path dependence.
- Future work: evaluate this data as a function of  $\Delta T$  and  $I_{OH}$

# Benefits of Operations with $T_{OH}$ up to 110 C

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# What Happens if the OH is Allowed to Operate up to 110 C?

## 1: Kinder Operating Window

### Both Simulations

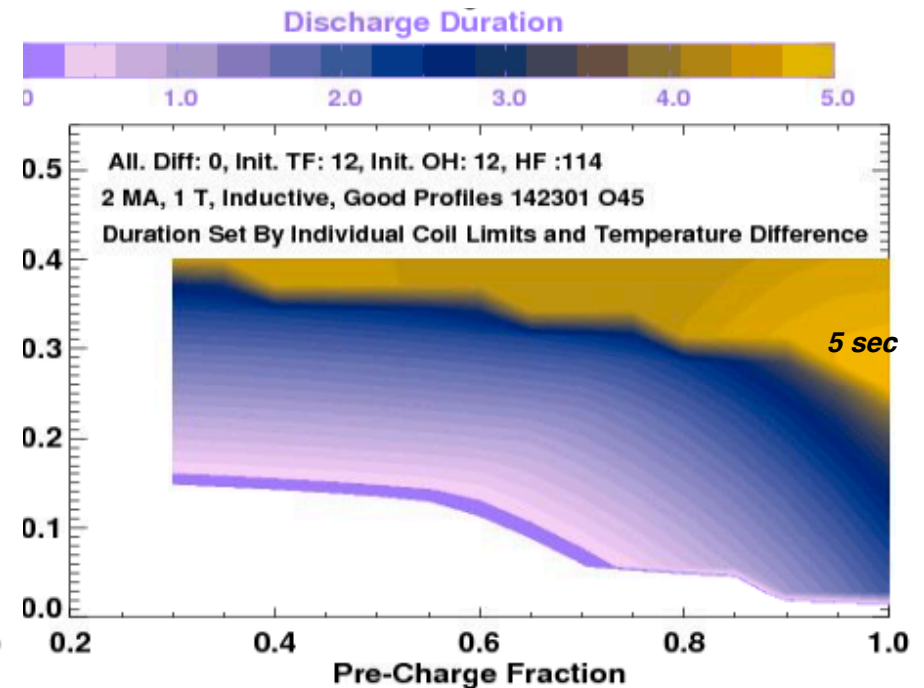
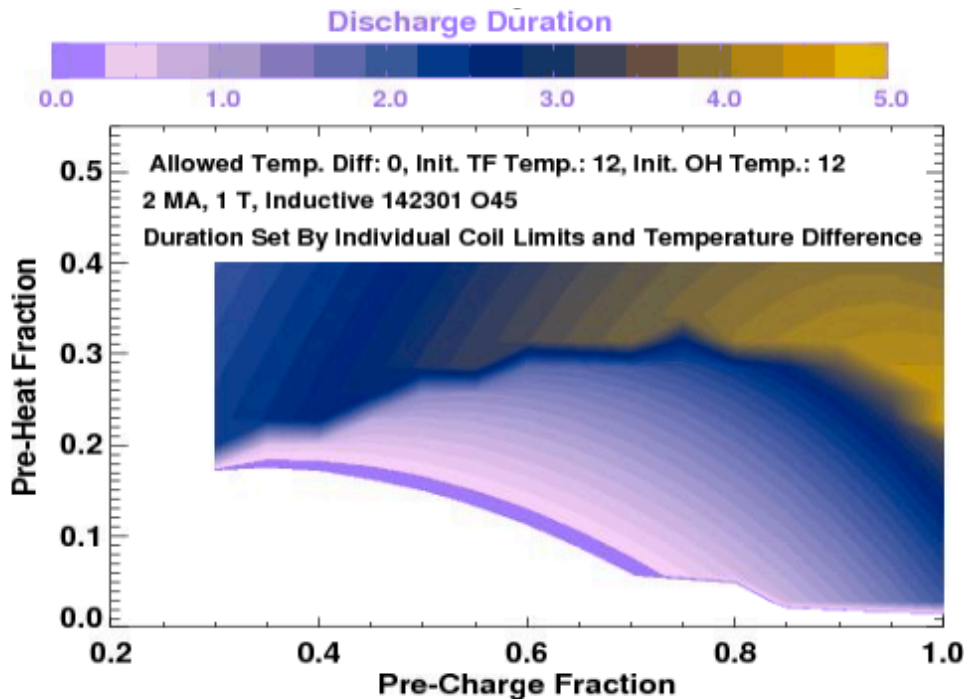
Initial Coil Temperatures of 12 C, with Resistive Pre-Heating Method

$T_{TF} < T_{OH}$  maintained

2 MA, 1 T, H~1.05, allows 5 sec. shot *without* relative temperature constraint

$T_{OH}$  limited to 98 C

$T_{OH}$  limited to 110 C



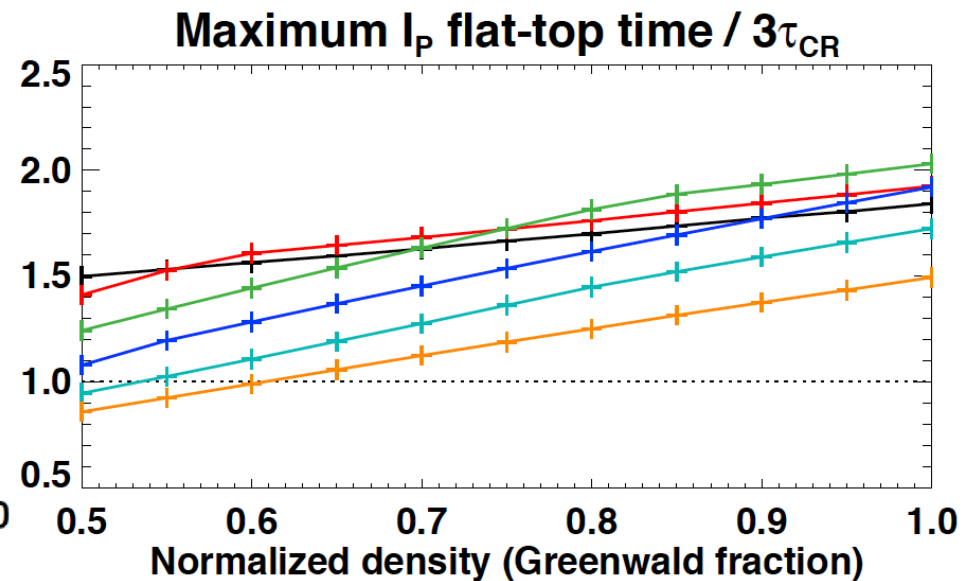
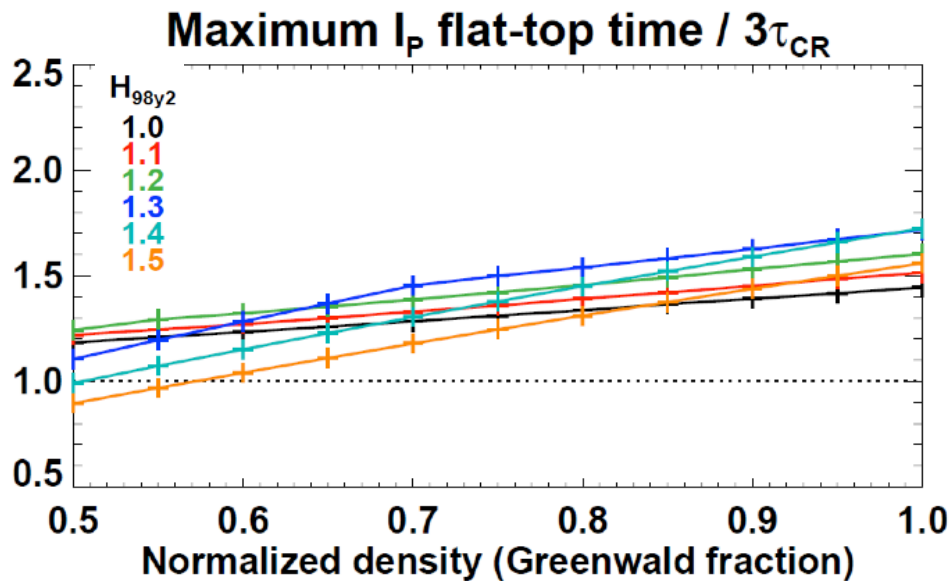
**For this confinement multiplier, 5 sec. operation restored with  
with 110 C max. OH temperature**

# What Happens if the OH is Allowed to Operate up to 110 C?

## 2: More robust access to $t_{\text{discharge}} > 3\tau_{\text{CR}}$

$T_{\text{OH}} < 100$  C,  $T_{\text{TF}} < 100$  C  
 Initial TF temperature: 12 C  
 OH Pre-heat to 43 C

$T_{\text{OH}} < 110$  C,  $T_{\text{TF}} < 100$  C  
 Initial TF temperature: 7 C  
 OH Pre-heat to 35 C



In this configuration, only 0.2-0.4 sec of absolute discharge duration lost, despite the **fixed initial OH temperature**

# Aquapour/CTD-425 Engineering Thoughts

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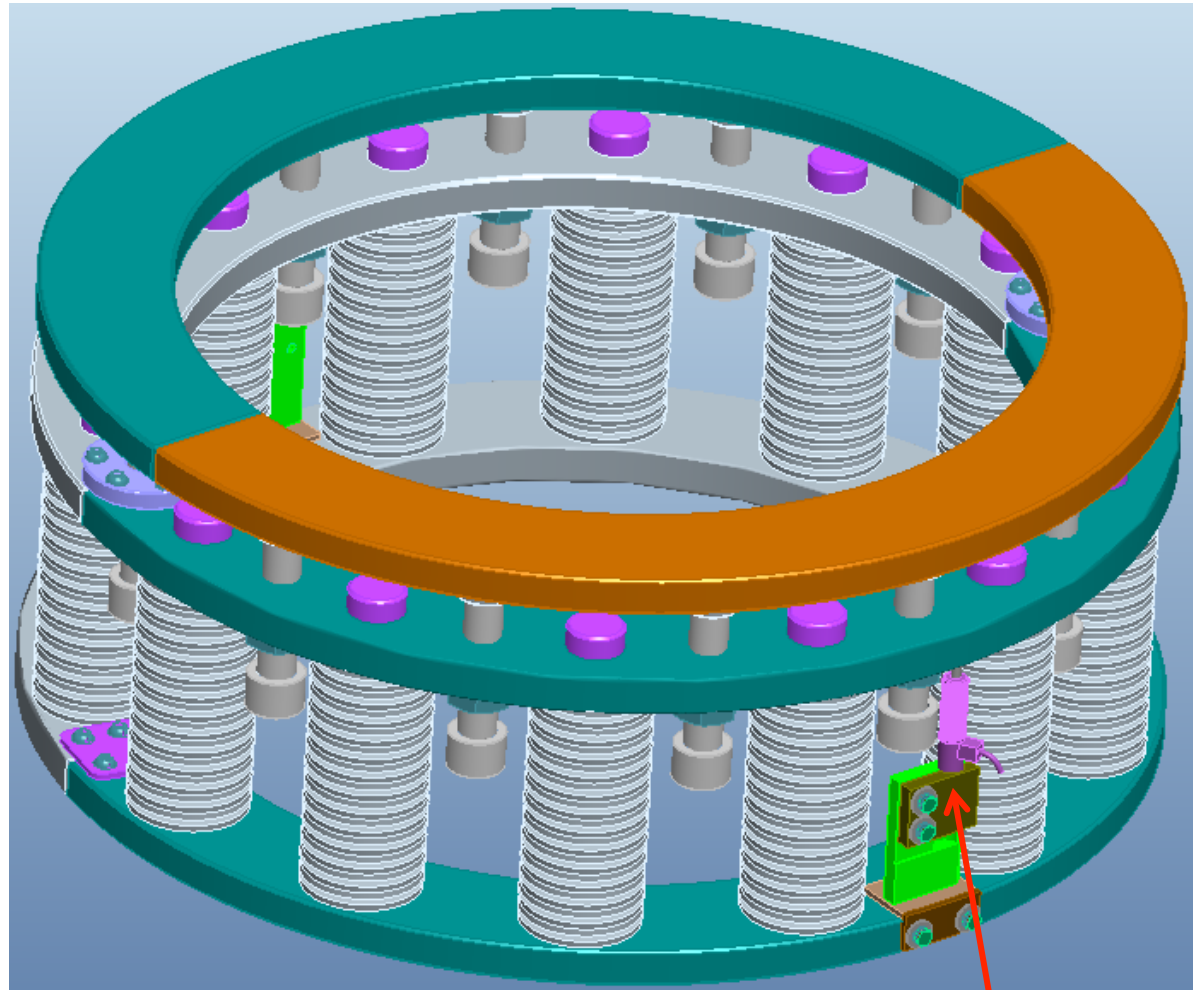
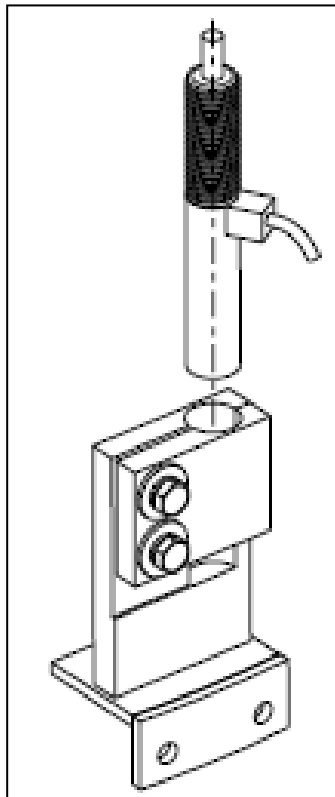
## DCPS Will Be Used to Enforce This Temperature Difference

- Operating engineer & water-systems PLC enforce that the coils be cooled to a pre-defined set-point at the start of the discharge.
- Coil temperature evolution computed in DCPS based on current measurements.
  - Compute the temperature difference at the  $i^{\text{th}}$  step:  $\delta T_i = T_{\text{TF},i} - T_{\text{OH},i}$
- Consider the heating that would occur in the event of a fault:
  - $\delta T_{\text{TF},\text{fault},i} = I_{\text{TF},i}^2 C_{\text{TF}}$ ,  $\delta T_{\text{OH},\text{fault},i} = I_{\text{OH},i}^2 C_{\text{OH}}$  (OH may or may not heat up more than the TF)
  - $\delta T_{i,\text{fault}} = (T_{\text{TF},i} + \delta T_{\text{TF},\text{fault},i}) - (T_{\text{OH},i} + \delta T_{\text{OH},\text{fault},i})$
- At each cycle, compare both  $\delta T_i$  and  $\delta T_{i,\text{fault}}$  to the defined limit (0 in the first year).
- Algorithm accounts for both instantaneous heating, and fault heating, while only relying on coil current measurements.
- Temperature evolution algorithm will be calibrated against outlet water temperature and potentially other measurements.

# OH Solenoid Thermal Growth Sensors Implemented

## FOD sensors will monitor OH solenoid growth

- Originally motivated by desire to monitor the pre-load.
- Two fiber optic displacement (FOD) sensors to be installed at 180° apart.
- The fixtures can be installed now and the sensors will be installed after the center stack is installed.



***Intent these to be used for trending data and analysis verification, not realtime protection***

**FOD Sensor**

# Permanent Aquapour/CTD-425 Composite Does Have Some Advantages

- OH coil will stay well centered on the TF bundle.
  - Eliminates the need for centering shims.
- OH pre-load mechanism is more robust.
  - OH pre-load provided by Belleville washer stack pushing on the TF coil flags.
  - 20 klb limit on the OH  $F_z$  determined by the hot-TF, cold-OH case.
  - By eliminating this case, the  $F_z$  limit is increased to 30 klb.
    - Provides additional headroom for control oscillations.

