

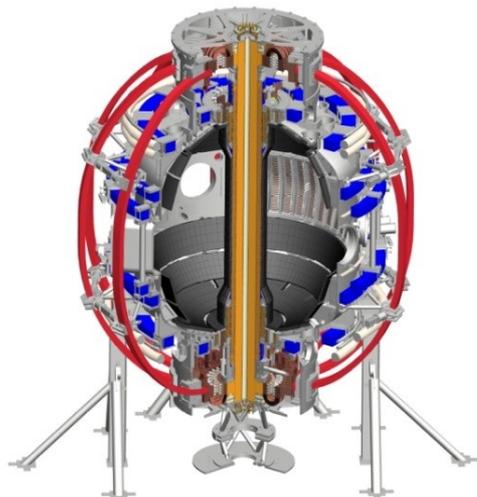
# Transition to Research on NSTX-U

## Stefan Gerhardt Research Staff

Head of Experimental Research Operations

**NSTX-U CD-4 Closeout  
B-318  
September 2<sup>nd</sup>, 2015**

*Coll of Wm & Mary  
Columbia U  
CompX  
General Atomics  
FIU  
INL  
Johns Hopkins U  
LANL  
LLNL  
Lodestar  
MIT  
Lehigh U  
Nova Photonics  
ORNL  
PPPL  
Princeton U  
Purdue U  
SNL  
Think Tank, Inc.  
UC Davis  
UC Irvine  
UCLA  
UCSD  
U Colorado  
U Illinois  
U Maryland  
U Rochester  
U Tennessee  
U Tulsa  
U Washington  
U Wisconsin  
X Science LLC*



*Culham Sci Ctr  
York U  
Chubu U  
Fukui U  
Hiroshima U  
Hyogo U  
Kyoto U  
Kyushu U  
Kyushu Tokai U  
NIFS  
Niigata U  
U Tokyo  
JAEA  
Inst for Nucl Res, Kiev  
Ioffe Inst  
TRINITI  
Chonbuk Natl U  
NFRI  
KAIST  
POSTECH  
Seoul Natl U  
ASIPP  
CIEMAT  
FOM Inst DIFFER  
ENEA, Frascati  
CEA, Cadarache  
IPP, Jülich  
IPP, Garching  
ASCR, Czech Rep*

# Outline

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- NSTX-U scientific goals
- NSTX-U test plasma results
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- Preparation for the first experimental campaign
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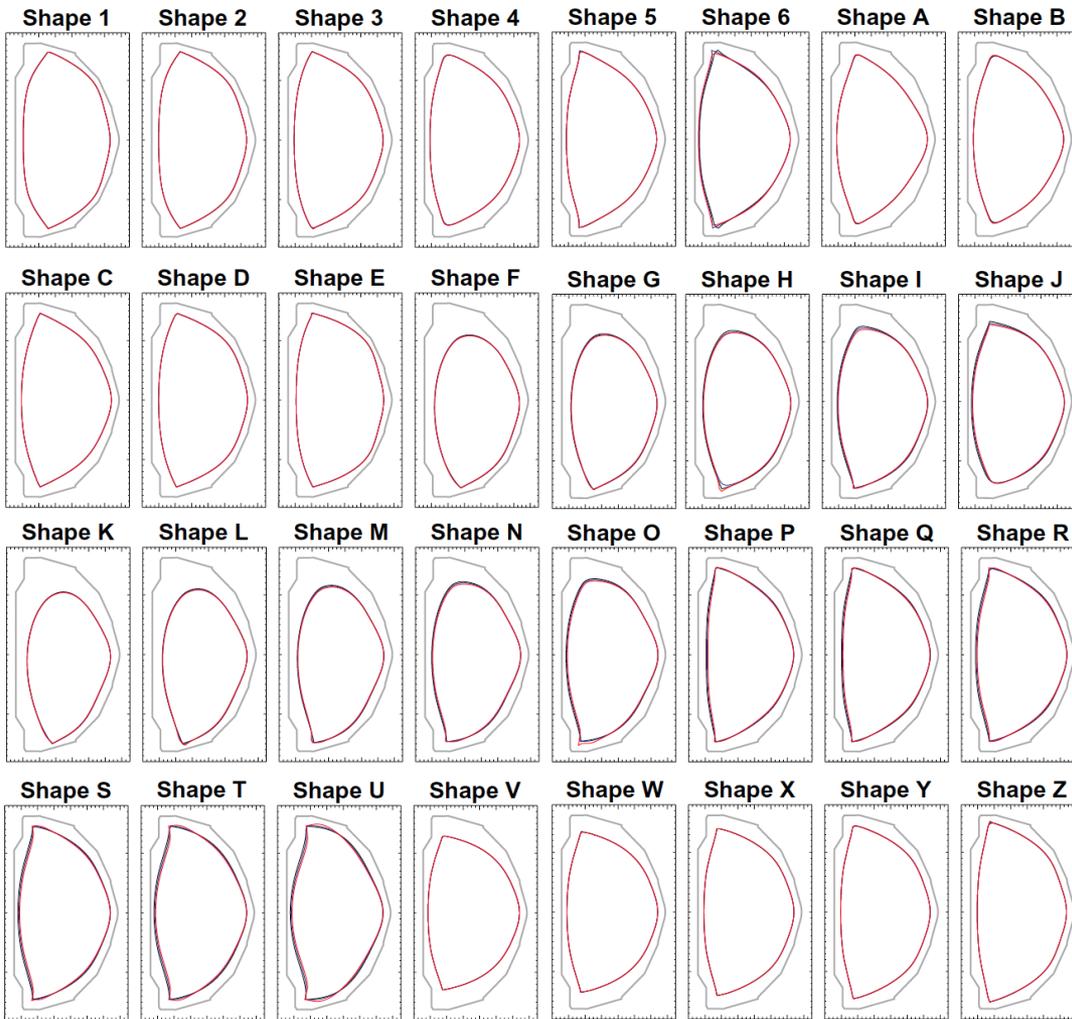
# Five Year Plan Described Five Highest Priority Research Goals

## Present Upgrade

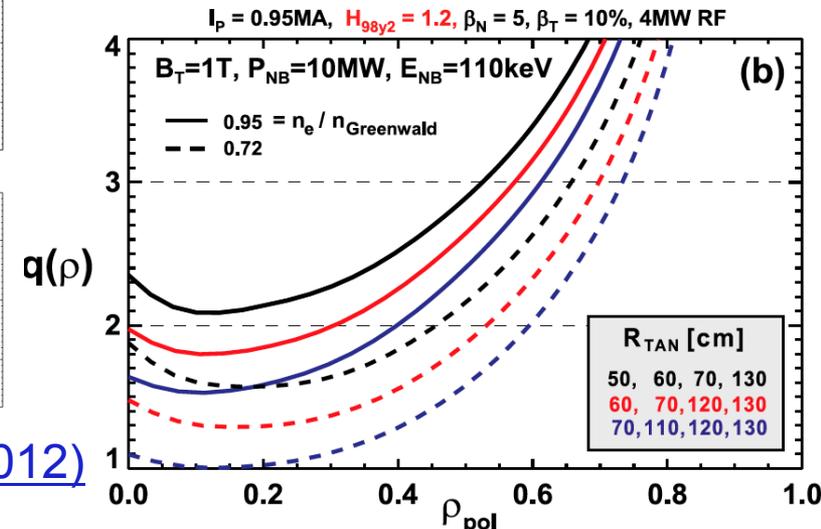
## Future Upgrade (See Backup Slides)

- 1. Demonstrate 100% non-inductive sustainment at performance that extrapolates to  $\geq 1\text{MW/m}^2$  neutron wall loading in FNSF**
  - 2<sup>nd</sup> neutral beam, higher TF
  - Cryopump (future upgrade) , NCC (future upgrade)
- 2. Access reduced  $\nu^*$  and high- $\beta$  combined with ability to vary q and rotation to dramatically extend ST physics understanding**
  - 2<sup>nd</sup> neutral beam, higher TF, higher  $I_p$
  - Cryopump (future upgrade), NCC (future upgrade)
- 3. Develop and understand non-inductive start-up and ramp-up (overdrive) to project to ST-FNSF with small/no solenoid**
  - 2<sup>nd</sup> neutral beam, higher TF
  - ECH (future upgrade)
- 4. Develop and utilize high-flux-expansion “snowflake” divertor and radiative detachment for mitigating very high heat fluxes**
  - Expanded PF-1 coil set, new divertor gas injectors
- 5. Begin to assess high-Z PFCs + liquid lithium to develop high-duty-factor integrated PMI solutions for next-steps**
  - Metal PFCs and flowing lithium systems (future upgrades)

# Engineering Design Driven By Physics Considerations



- Poloidal field set consistent with a set of 96 plasma equilibria at 2 MA, 1T.
  - Defines coil locations, power supply requirements, forces/stresses on the coils & vessel, some DCPS limits
- Beam tangency radii determined by integrated modeling with TRANSP.
  - Provides a range of current profiles.



J. E. Menard, et al., Nuclear Fusion **52**, 083015 (2012)

# Long-Term Research Agenda For NSTX-U is Defined in the 5-Year Plan

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- Available on the web at:
- <http://nstx-u.pppl.gov/five-year-plan/five-year-plan-2014-18>
- 11 Chapters, written by the entire NSTX-U team, describing
  - the research goals
  - future upgrades to the facility
- Reviewed over three days in May 2013.
- Accepted by DoE.

# Time-Scales Involved in this Talk

- Energy confinement time: 0.04-0.1 seconds
  - The representative time-scale for energy to leak out.
- Current redistribution time: 0.2-1.5 seconds
  - The representative time scale for current to penetrate
- Expected discharge duration: 2-5 seconds
  - Set by many plasma physics and facility constraints
- Discharge repetition rate: 15-30 minutes
  - Set by OH coil or TF buss work cooling
  - Number of discharges in a day: 17-35 discharges
- Run campaign: 12-16 weeks
  - May take 16-20 calendar weeks to accumulate this time

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# CS KPP Run-Up

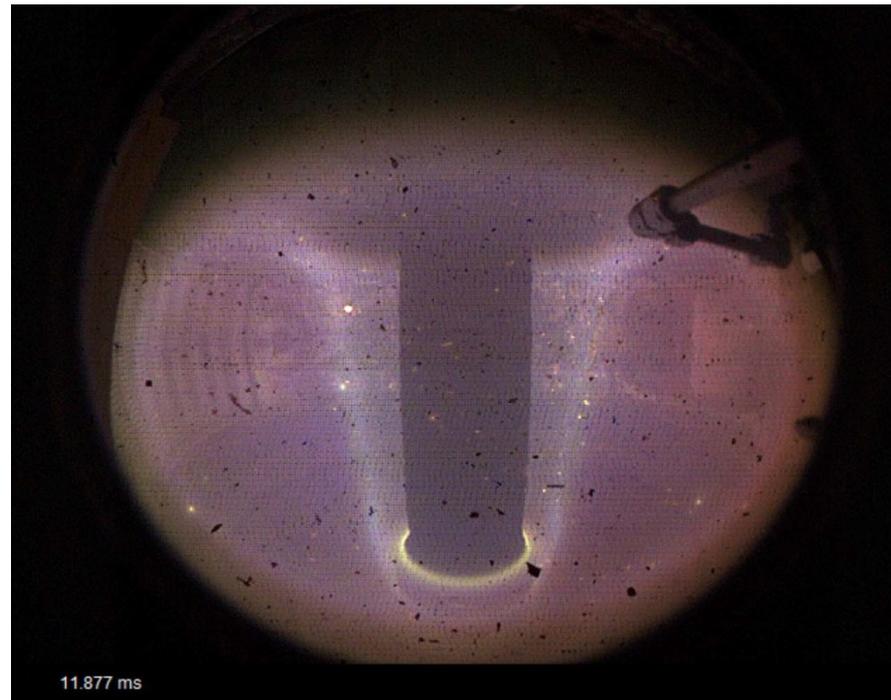
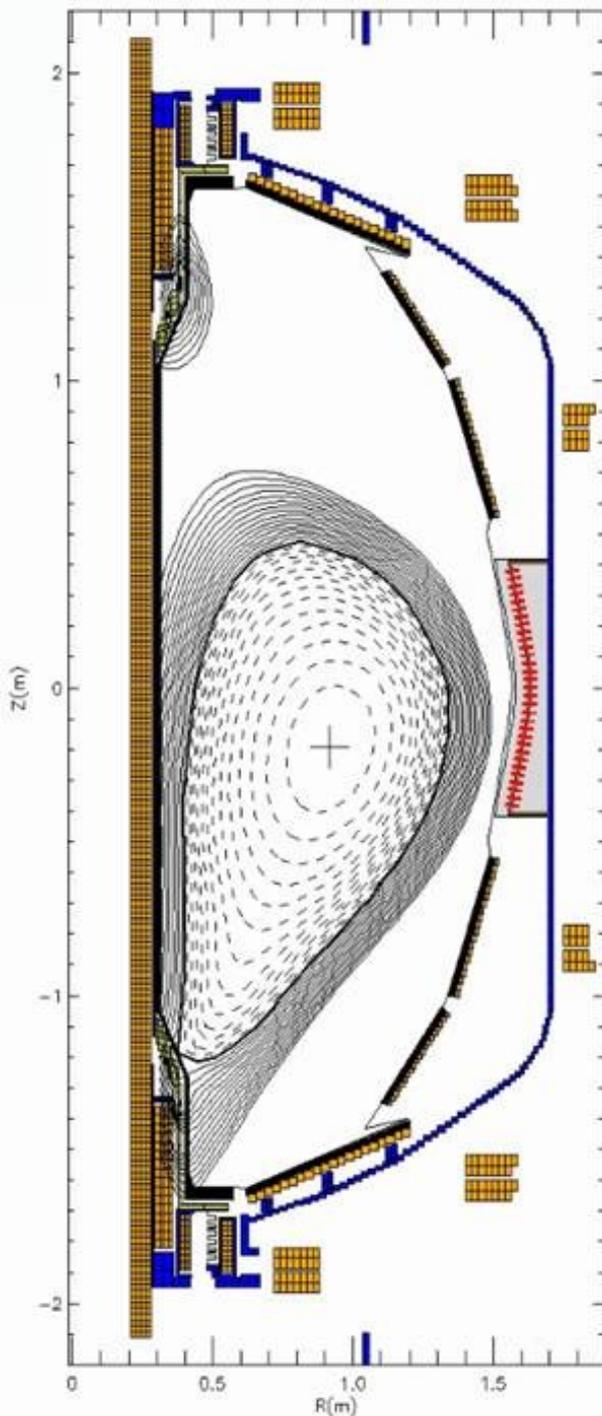
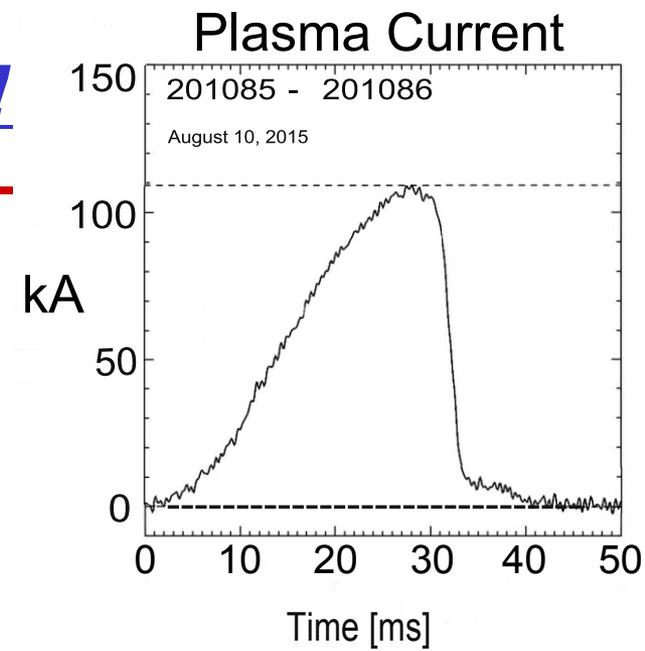
- [8/3/2015] ES&H Executive Board accepted the ACC recommendation to restart the facility.
- [8/4/2015] Begin the coil system Integrated Systems Test Procedure (ISTP-001)
  - [8/4/2015] Complete Coil High-Pots
  - [8/5/2015-8/7/2015] Single Coil Test Shots
  - [8/10/2015] Combined Field Test Shots
- [8/10/2015] Begin plasma operations under XMP-100.
  - Achieved ~110 kA of plasma current
- [8/11/2015 & 8/12/2015] Continued operation on XMP-131.
  - Achieve ~140 kA, improve plasma positioning.

# !! 110 kA Plasma !!

Measured Plasma Current

EFIT Reconstruction of the Plasma Shape

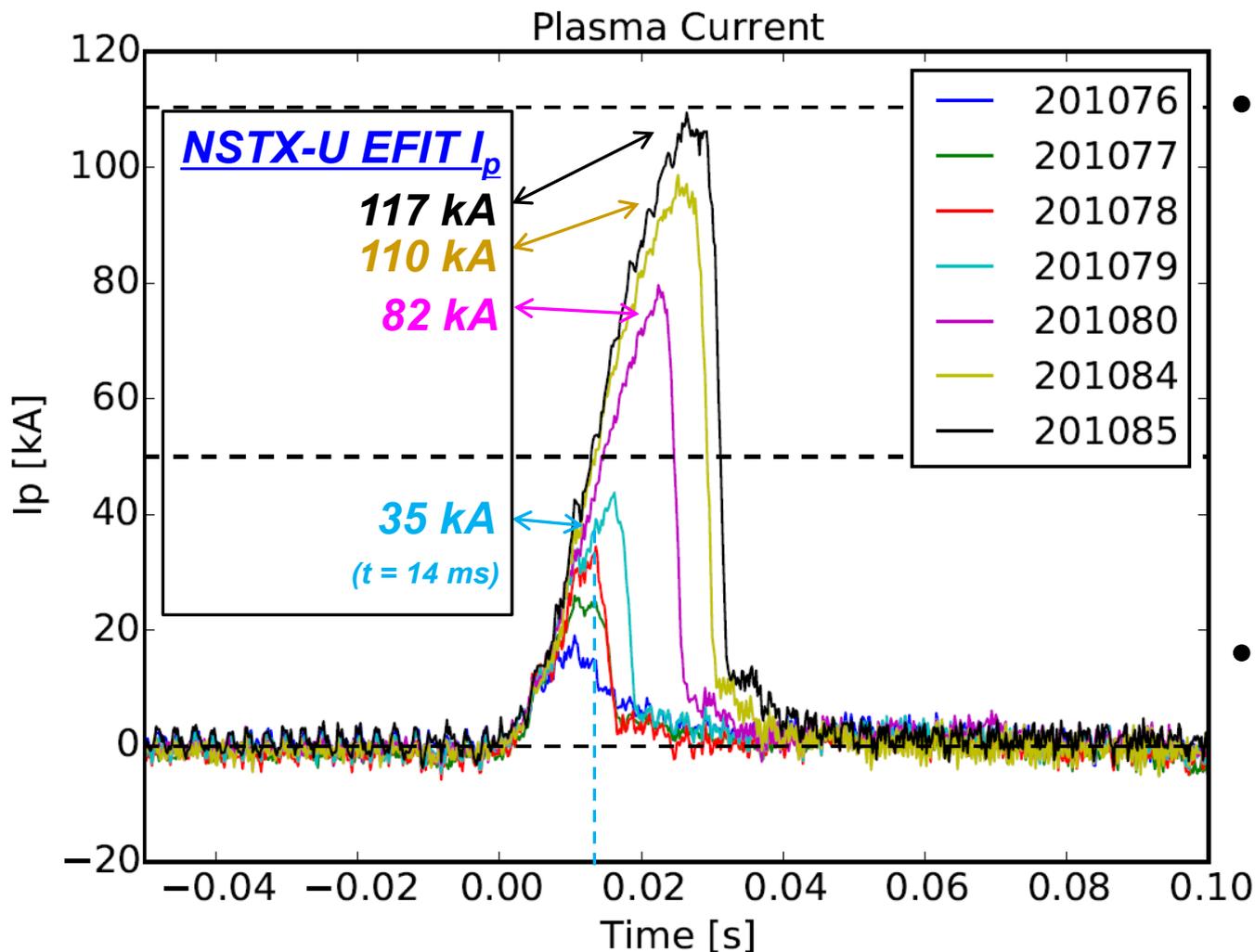
[S. Sabbagh  
Columbia University]



Fast Color  
Camera  
Image  
[F. Scotti, LLNL]

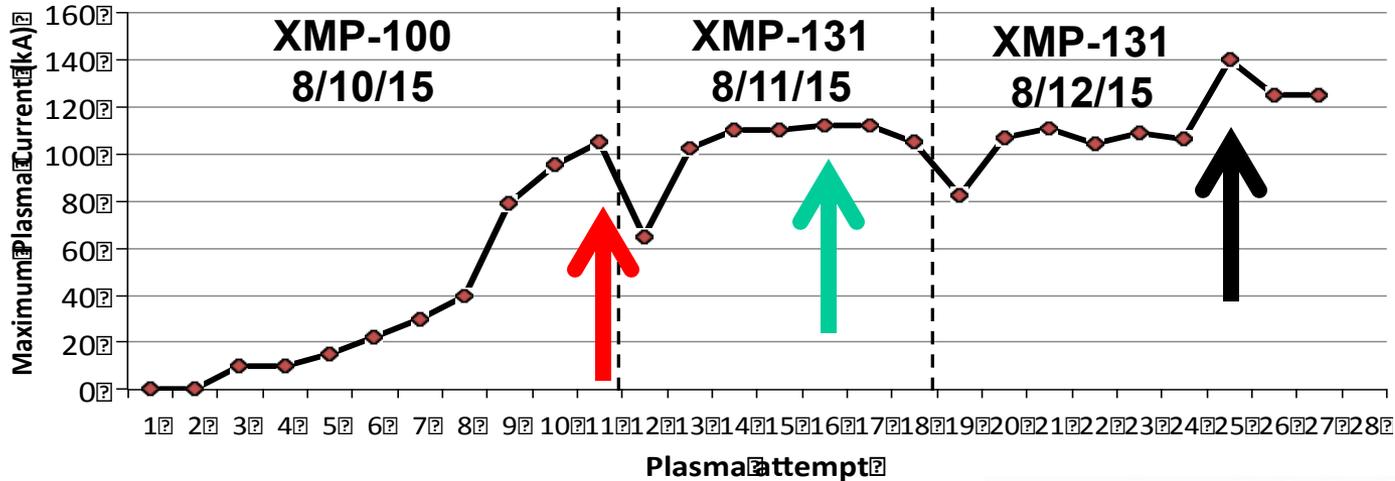
# Measured, compensated plasma current compares well to NSTX-U EFIT reconstructed current

Shots from 8/10/2015



- In highest  $V_{loop}$  shots, reconstructed total wall current exceeds 0.4 MA
- Achieved highest current on last shot of day.

# Continued Improvement in Plasma Current and Duration in Sixteen Plasma Shots over 1.5 Days

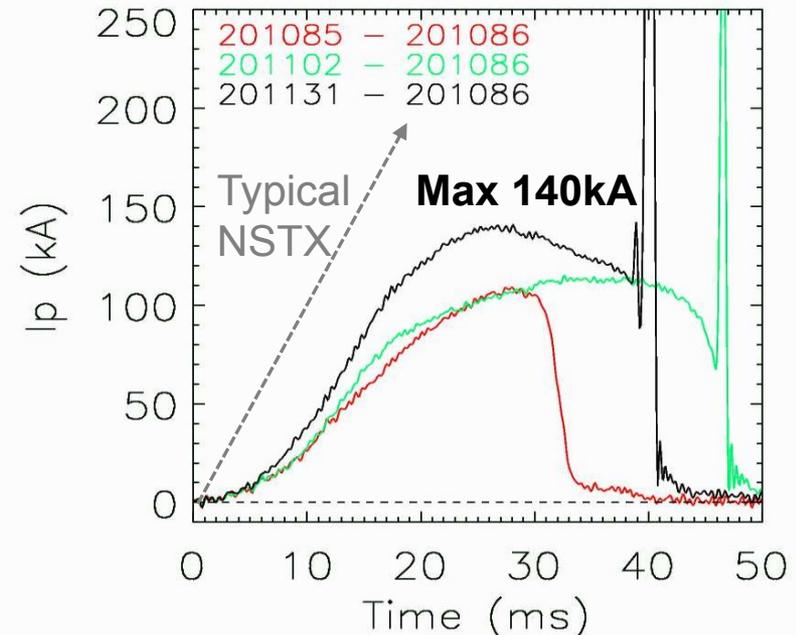


(27 out of 29 attempts were "good" over 2 days)

(First shot after morning He glow takes a hit)

- Centering plasma and shrinking outer gap lengthened discharge
- Reducing prefill fueling increased  $I_p$  ramp rate (Wed)

Performance Should be Dramatically Better Once We do the Full Vessel Bake

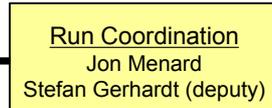
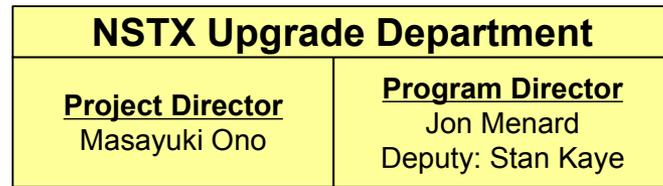


# Outline

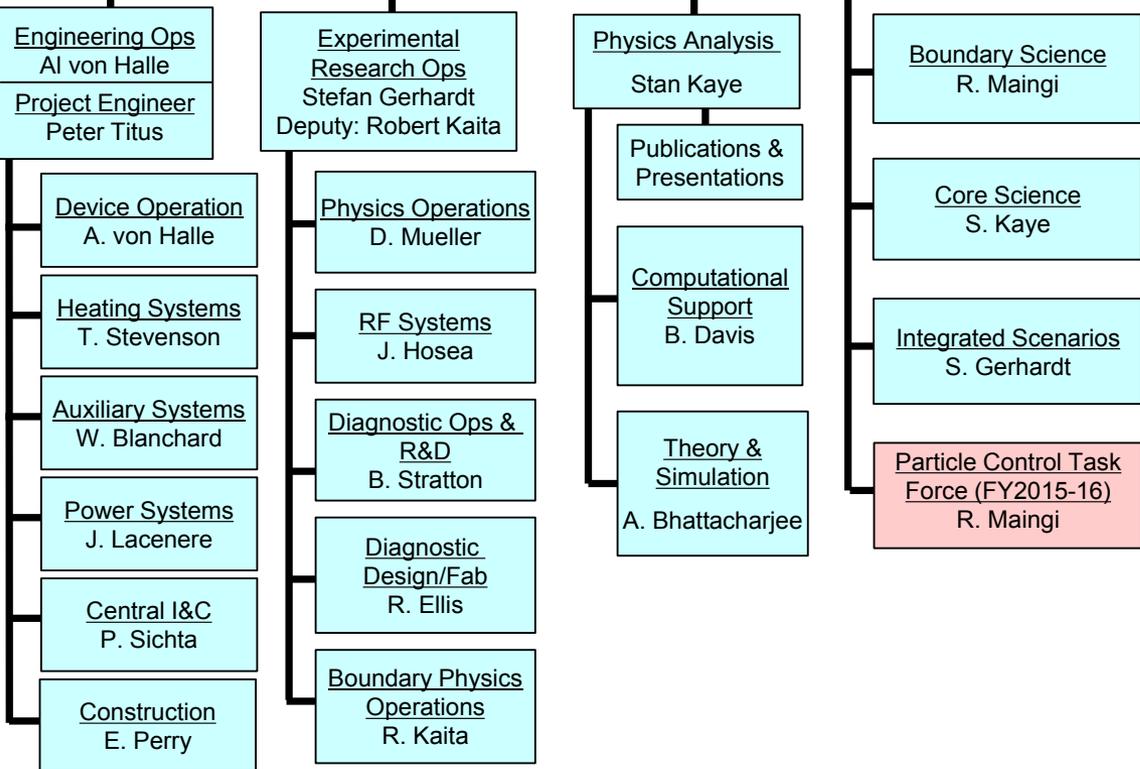
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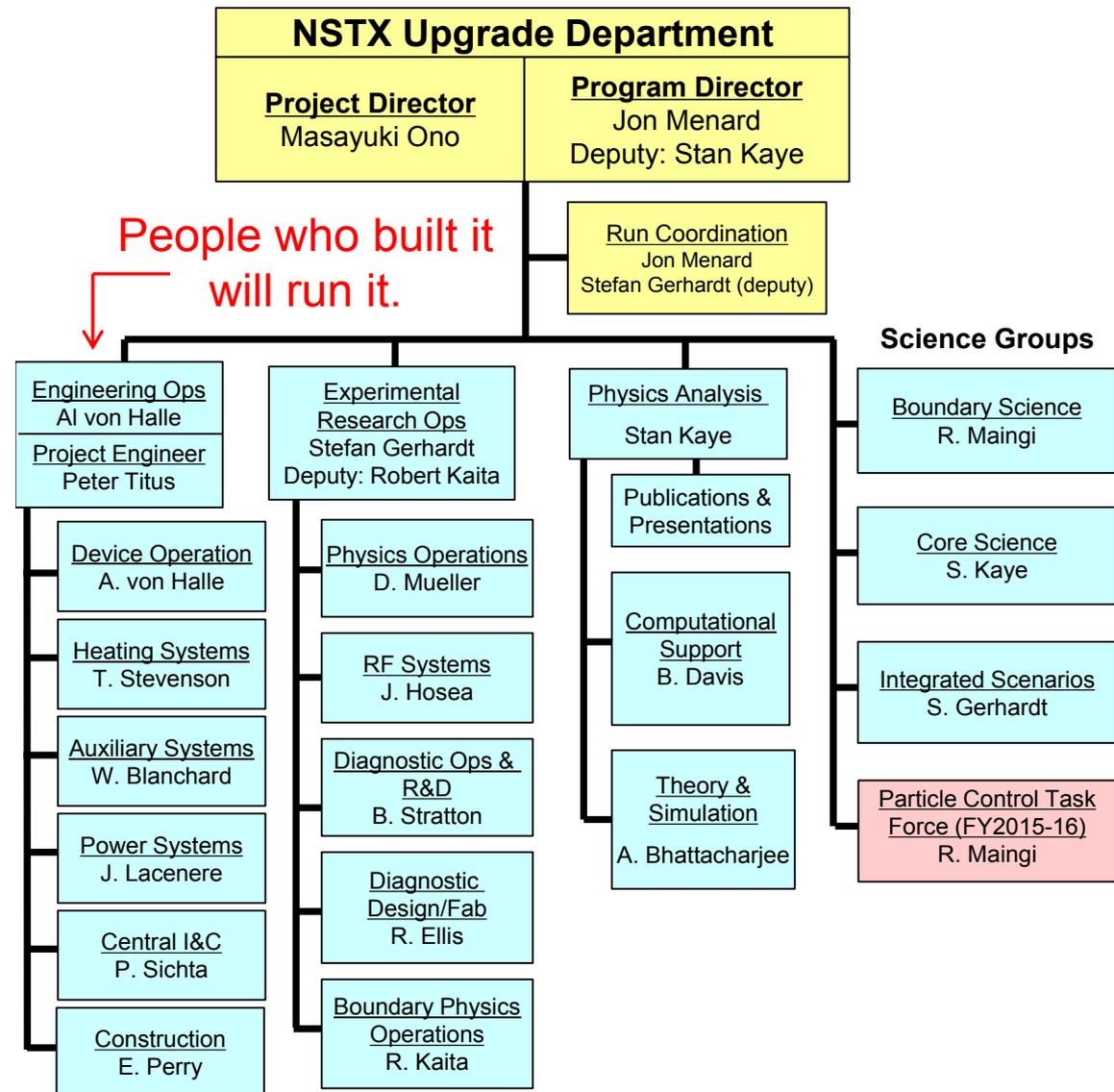
# NSTX-U Organizational Structure is Clearly Defined



## Science Groups



# NSTX-U Experimental Structure is Clearly Defined

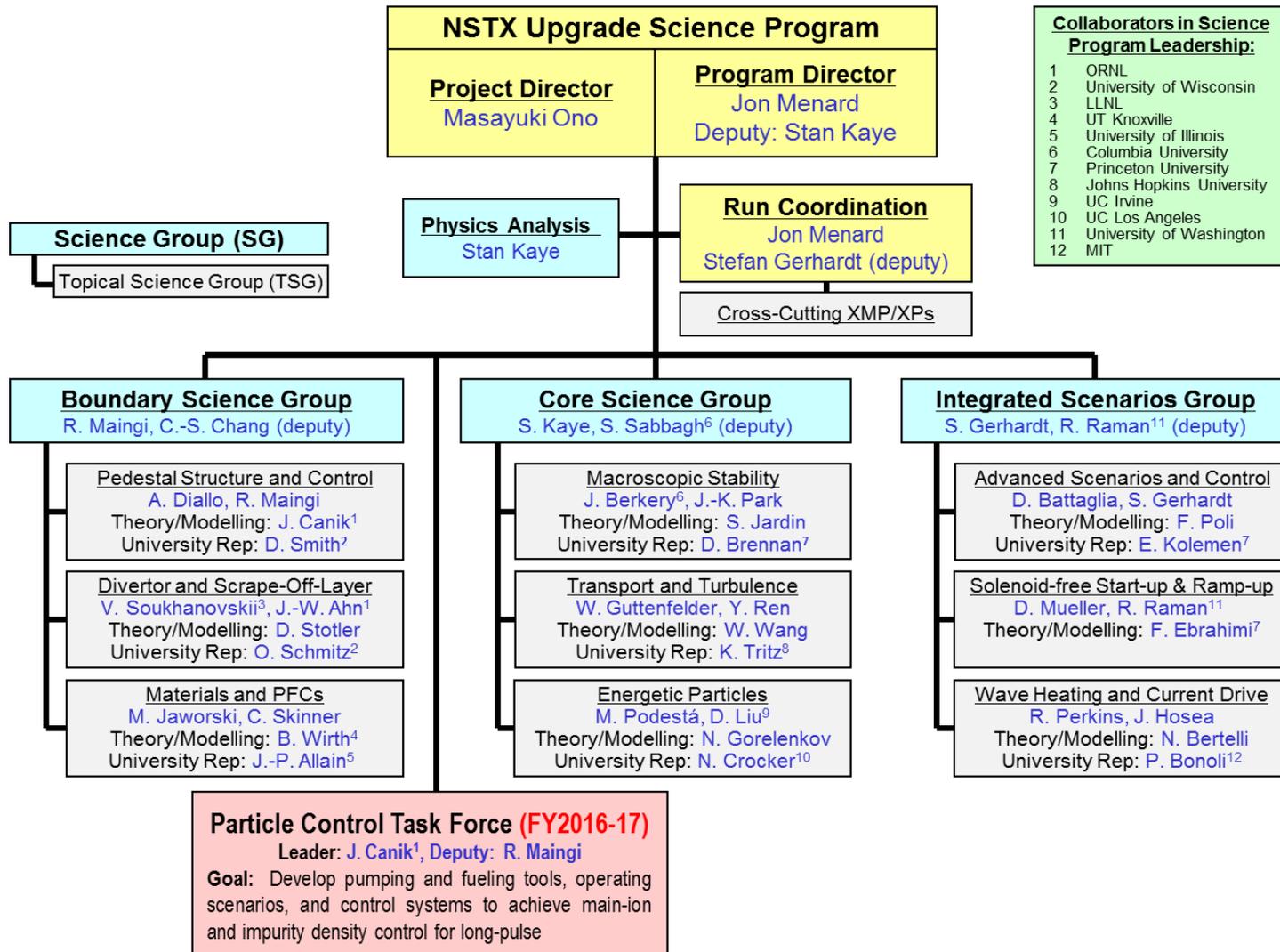


This structure defines

- i) science program, &
- ii) engineering/operations structure to execute the activities.

Actual design, fabrication, & construction, and operations activities accomplished by PPPL engineering via their procedures and processes.

# NSTX-U Research Program Is Organized Along 3 “Science Groups” and 9 TSGs for the FY16 run



er, deputy,  
iversity rep  
ticipation

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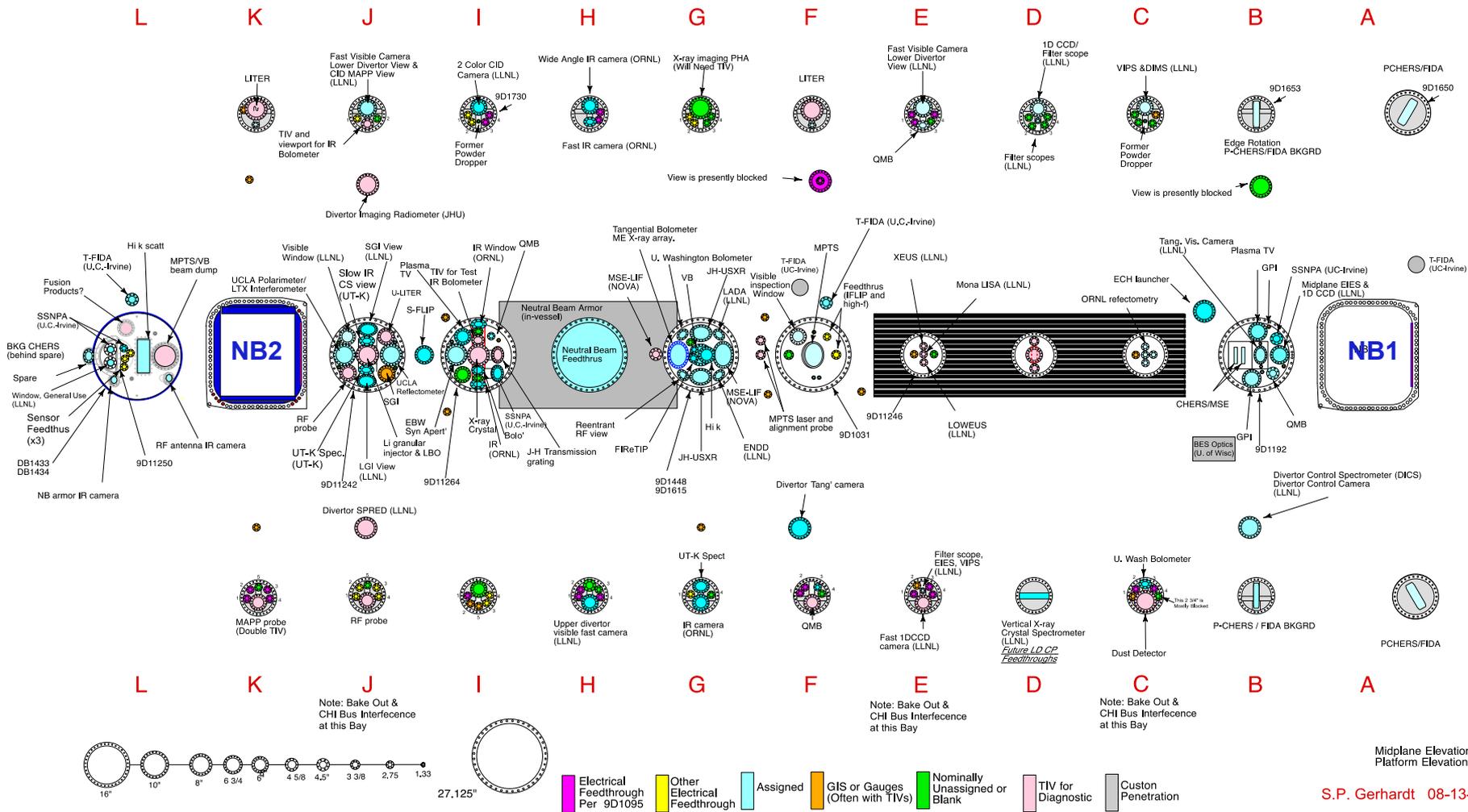
# Many “Non-Upgrade” Tasks Have Been Undertaken to Prepare for the Run

- Crucial diagnostics
  - Many upgrades to the magnetic diagnostics
  - Large changes to the critical Thomson scattering systems successfully implemented...Rayleigh/Raman scattering complete!
  - All major profile diagnostics installed and calibrated.
  - Many new or upgraded diagnostics...
- Upgrades to the High Harmonic Fast Wave (HHFW) antenna.
- New boronization systems with improved safety features.
- New rectifier control hardware, new plasma control computers and many algorithm upgrades

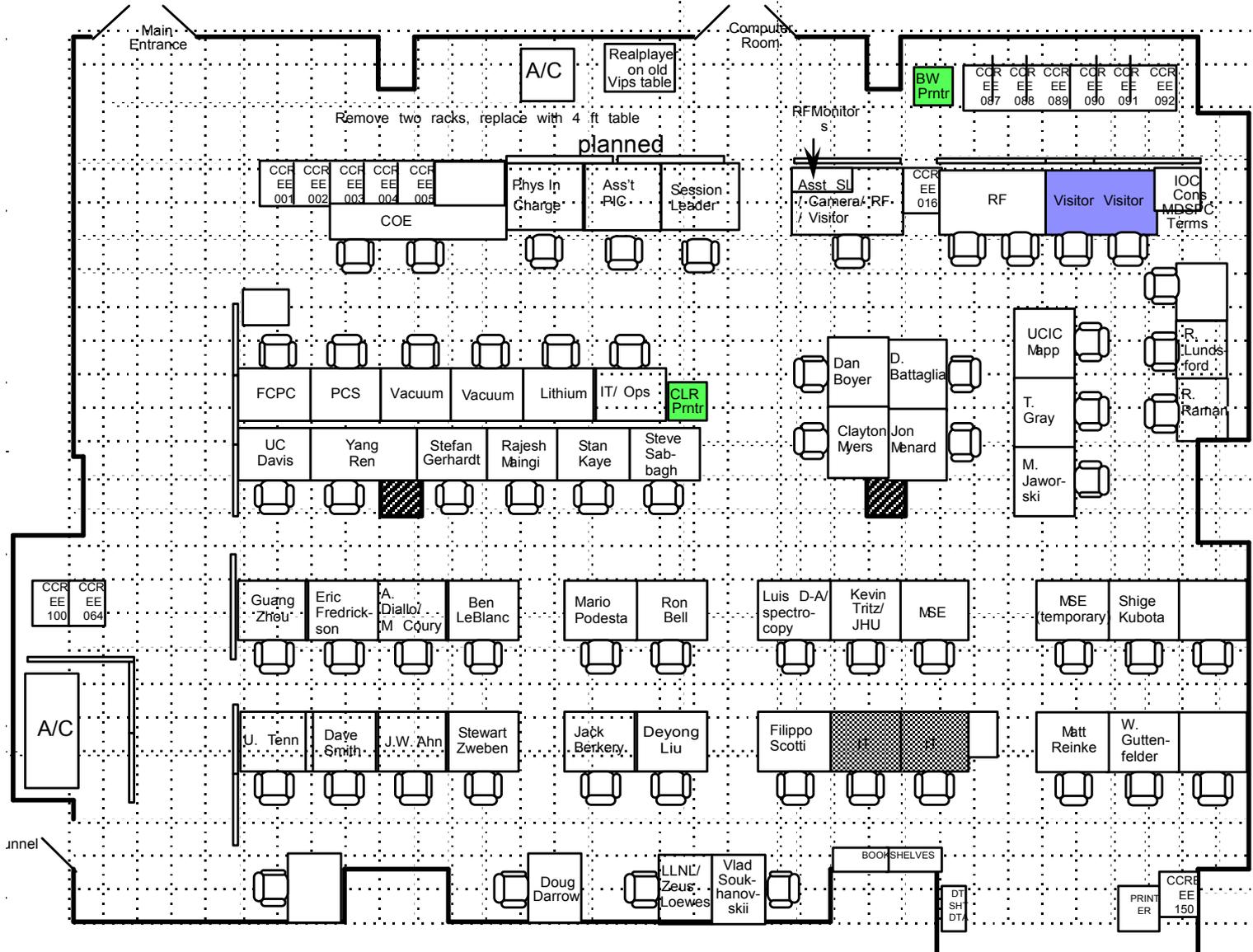
***Many details in backup...***

# Most Previous Diagnostics Reinstalled or Relocated, Many New Diagnostics Installed

## Gas Injectors and Ion Gauges on Separate Port Drawing



# Seat Assignments in the Control Room Have Been Updated



## We Have Three Research Milestones +JRT in FY-16

- R16-1: Assess H-mode confinement, pedestal, SOL characteristics at higher  $B_T$ ,  $I_P$ ,  $P_{NBI}$
- R16-2: Assess effects of NBI injection on fast-ion  $f(v)$  and NBI-CD profile
- R16-3: Develop physics + operational tools for high-performance:  $\kappa$ ,  $\delta$ ,  $\beta$ , EF/RWM
- JRT: Assess disruption mitigation, initial tests of real-time warning, prediction

See: <http://nstx-u.pppl.gov/program/milestones>  
Full 3-year set of milestones in backup

# Daily Operations Directed by Experimental Proposals (XPs) and Experimental Machine Proposals (XMPs)

## XPs

- Describe experimental steps to answer science questions
- Governed by OP-ADX-03
- Reviewed by
  - topical science group
  - run coordinator + research team
- Typically described 1/2-2 days of machine operations
- Expectation that each XP will lead to a publishable result

## XMPs

- Describe experimental steps to qualify new machine capabilities
- Governed by OP-ADX-02
- Reviewed by
  - physics operations branch head
  - research operations division head
- Typically describe 1/2 -1 day of machine operations
- Expectation is that each XMP will facilitate multiple XPs

# XPs and XMPs for FY-16 Defined at the Research Forum, and Subsequently Further Refined

- Research Forum was held at PPPL Feb. 24<sup>th</sup>-27<sup>th</sup>
  - Requests made for ~270 days of run time, from 84 different first authors, for ~80 available days.
  - Initial prioritizations performed within TSG and SG breakout sessions.
  - Initial XP & XMP sequencing defined.
- Now in the process of reviewing and finalizing XPs and XMPs.
  - 6 XMPs have been approved.
  - 17 XPs have been reviewed by run coordination, 8 approved.



# We Plan to Increase the Field, Current & Pulse Duration Over 3 Years

	NSTX (Max.)	FY 2016 NSTX-U Operations	FY 2017 NSTX-U Operations	FY 2018 NSTX-U Operations	Ultimate Goal
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Allowed TF $I^2t$ [MA <sup>2</sup> s]	7.3	80	120	160	160

1<sup>st</sup> year goal: operating points with forces up to ½ the way between NSTX and NSTX-U, ½ the design-point heating of any PF/TF coil (~75% for OH)

2<sup>nd</sup> year goal: Full field and current, but still limiting the PF/TF coil heating

3<sup>rd</sup> year goal: Full capability

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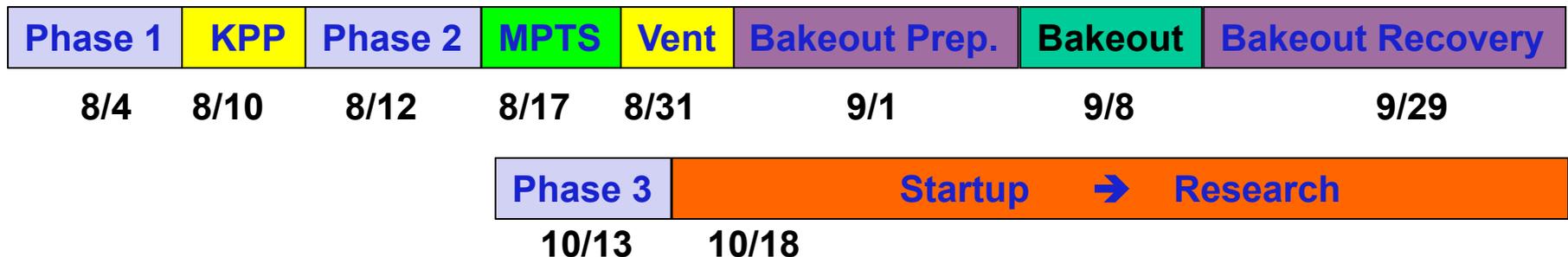
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# Sequence From Test Plasma To Full Research is Well Defined

- Phase 1 Coil Testing
  - Commission TF, OH, PF coil systems required for test plasma.
- CS KPP
- Phase 2 Coil Testing:
  - Commissioned remaining coil systems
- MPTS Rayleigh-Raman Scattering
- Small Vent
- Bakeout
- Phase 3 Coil Testing
  - Prepare for Commissioning/Startup Phase
- Commissioning/Startup Phase
- Research Ops



*You are Here!*



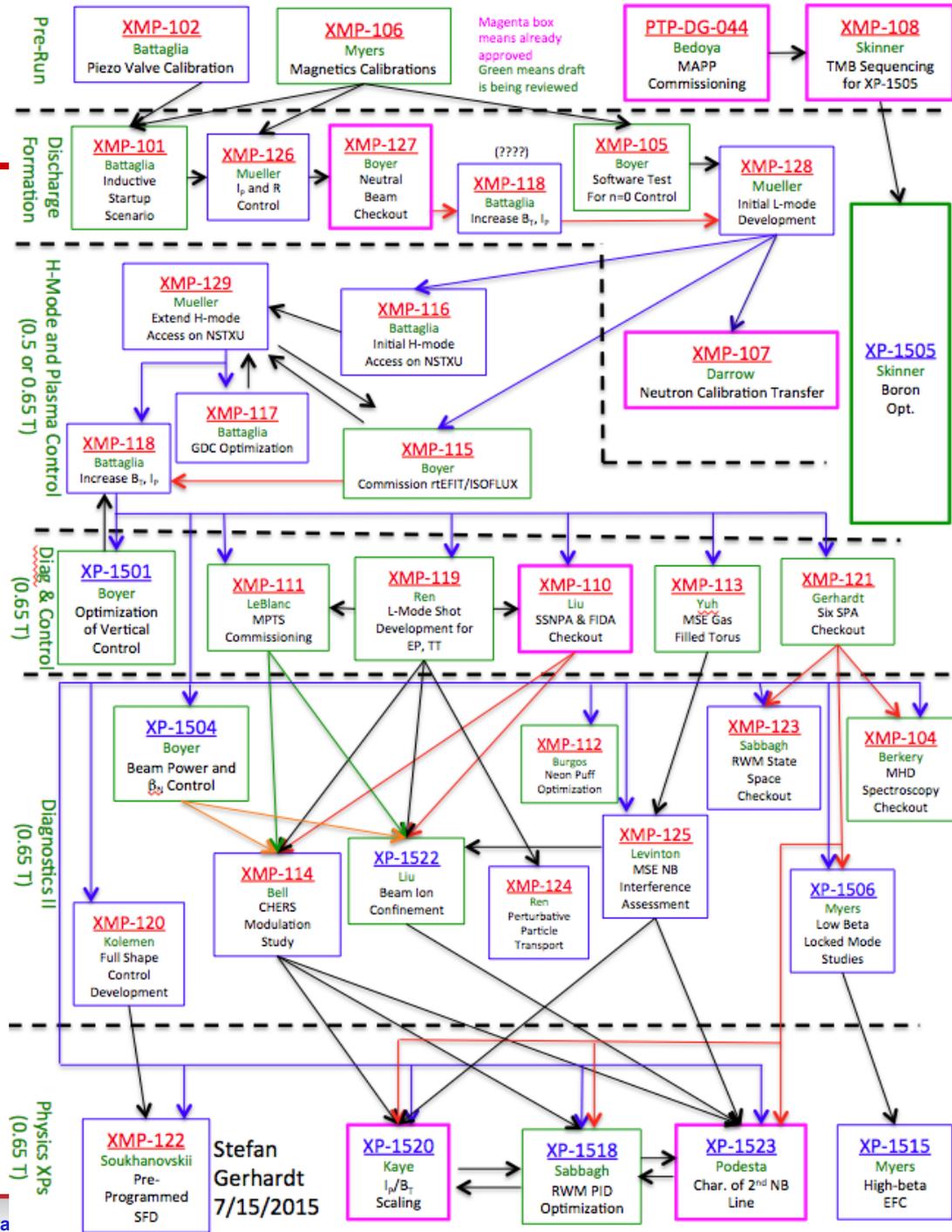
# The NSTX-U Research Program Will Initiated By a Sequence of XMPs

- XMPs for pre-plasma calibrations (3)
- XMPs to reestablish basic “L-mode” plasma operations (7).
- XMPs for “H-mode” access and advanced plasma boundary control (5)
- XMPs for additional control development and initial diagnostic checkout (6)
- XMPs for advanced diagnostic checkout (6)

***At the completion of this list, we will be ready to execute the critical XPs ( $I_p$  &  $B_T$  Scaling, Characterization of the 2<sup>nd</sup> NB Line)***

# The Linkages of the First XMPs and XPs Have Been Defined

- Obviously, not going to discuss this in detail.
- Engineering requirements are consistent with present facility plans.
  - 2 beam boxes
  - Full set of TF, OH, PF coils
    - TF to 0.65 T.
  - 6 SPAs (for RWM coils)
  - Boronization



# Physics Operations Staff+Collaborators Will Be Ready to Execute the NSTX-U Research Program

- Three NSTX physics operators will return to NSTX-U.
  - D. Mueller is a world-recognized tokamak driver.
    - Operated TFTR
    - NSTX and NSTX-U Physics Operations Branch Head.
    - Has collaborated on EAST and K-STAR control development over the last years.
  - D. Battaglia has spent the last 2 years as a DIII-D physics operator
    - Was responsible for the test plasma XMPs
  - R. Raman (U. of Washington) provides leadership in CHI, MGI areas + physics operations.
- D. Mueller holding a physics operator course.
  - 1/2 in July, 1/2 in September.
  - Plan to train an additional 2-3 physics operators.
  - Slides for course:
    - [http://nstx.pppl.gov/DragNDrop/Operations/Physics\\_Operations\\_Course/](http://nstx.pppl.gov/DragNDrop/Operations/Physics_Operations_Course/)
- Major diagnostics have primary and backup support.

# Operating Period Likely Runs Through April, in Order to Achieve 14-16 Run Weeks

- If FY16 budgets are favorable enough, may run more run weeks
- Want as much data as possible for IAEA synopses/meeting, APS-2016

- October: 2 run weeks (XMPs)
- November: 2 run weeks (XMPs → XPs)
  - May want to slow/pause for ST workshop (Nov. 3-6), APS (Nov 16-20), Thanksgiving (Nov. 26)
- December: 3 run weeks
- January: 2-3 run weeks
  - Mid-run assessment (if applicable), PAC-37
- Feb-Apr: 6-8 run weeks, complete FY16 run
- Apr/May: Start outage: install high-k, high-Z tiles, ...
- Resume operations fall/winter 2016 for FY17

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## Summary: NSTX-U is Well on the Way To an Exciting First Run Campaign

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- Upgrade was designed to facilitate the research program.
  - And successfully built (Ron's talk).
- The test plasma activity was very successful.
- The scientific team, and management team, are in place to develop and exploit the facility.
- The sequence of events leading to research operations is well defined, and we are well along the way.

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# Backup

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# Operations-Side Facility Enhancements

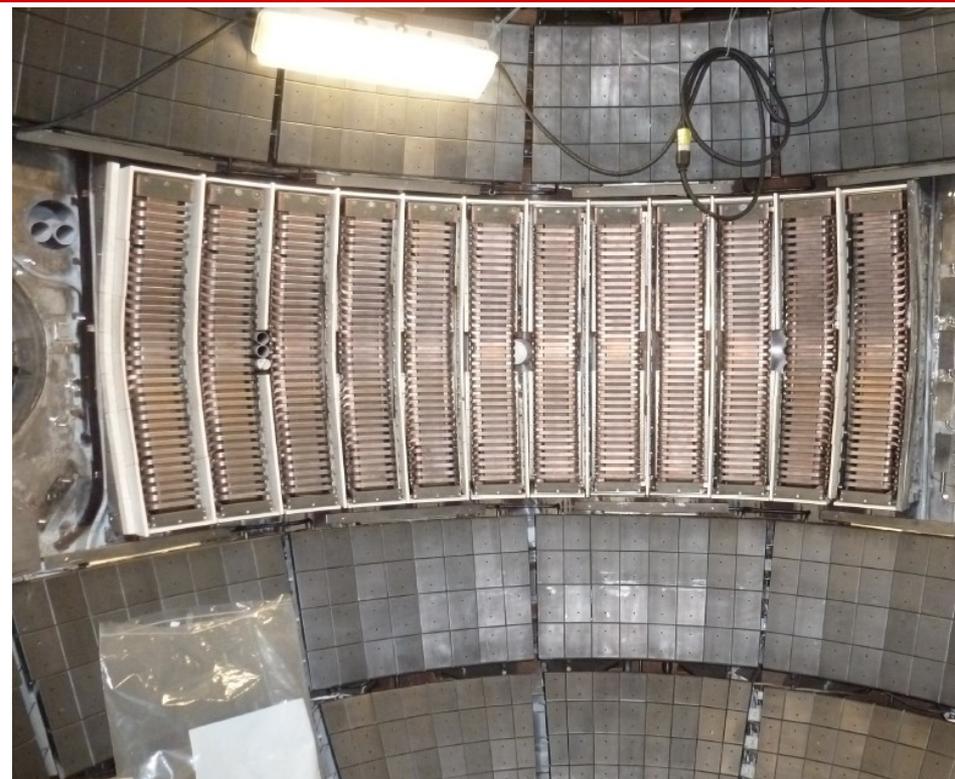
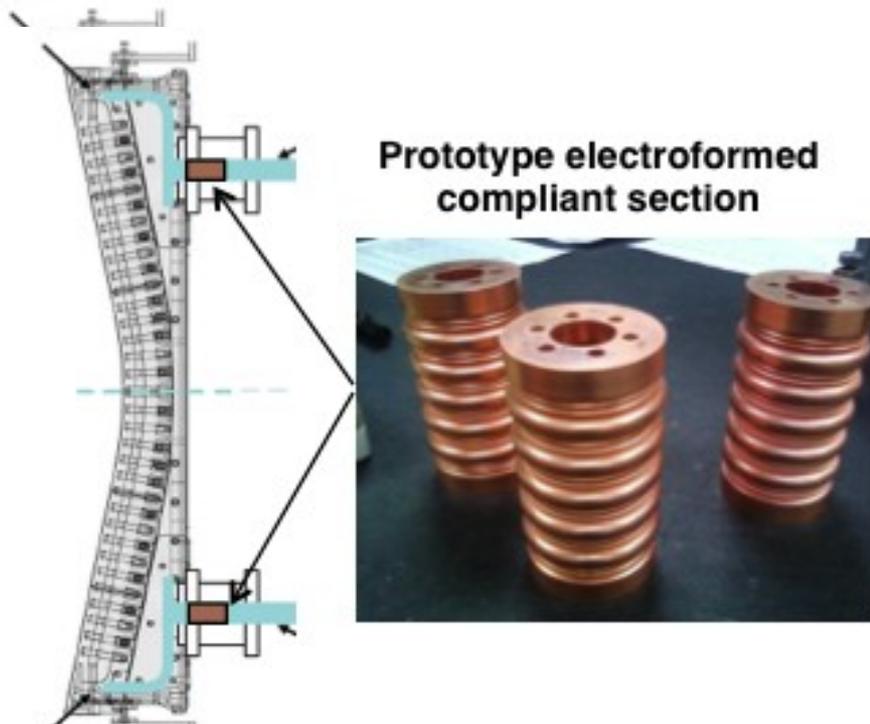
# Operations-Side Activities to Support Operations:

## Plasma Diagnostics

- Thomson scattering ready for operations:
  - Vessel modifications
  - New flight tubes
  - Rayleigh/Raman scattering is complete
- Neutron detectors installed, and calibrated with in-vessel train track.
- New diagnostics in final stage of installation:
  - Three new SSNPAs (UC-Irvine)
  - Bolometer, multi-energy SXR system (PPPL & JHU)
  - EUV spectroscopy systems (LLNL)
- Other major profile diagnostics reinstalled, spatially calibrated, intensity calibrated where appropriate:
  - CHERS, FIDA, T-FIDA, P-CHERS, MSE, MSE-LIF
- Magnetic diagnostics on CS expanded as part of the upgrade project.
  - And legacy magnetics on the outer vessel tested, repaired.
- Three new large port covers fabricated, installed, populated to accommodate systems displaced by vessel modifications.
- Considerable activity now being spent on special purpose diagnostic development.

# Operations-Side Activities to Support Operations: HHFW Heating and Current Drive System

**New Compliant Antenna Feeds**  
Will allow HHFW antenna feedthroughs  
to tolerate 2 MA disruptions



Prototype compliant feeds tested to 46 kV in the RF test-stand. Benefit of back-plate grounding for arc prevention found. Antennas were re-installed with the new feeds and back-plate grounding

- Transmission lines have been installed and tuned
- 4 of the 6 transmitters have been tested into dummy load.
- Final 2 transmitters are being brought up now.

# Operations-Side Activities to Support Operations: Physics Operations & Plasma Control

- New firing generators for transrex recifiers designed, tested, fabricated, deployed.
- New in-house design for realtime digitizer designed, tested, fabricated, ready for deployment.
- New  $I_p$  measurement systems and associate permissive generators designed, fabricated, deployed.
- Plasma Control System (PCS)
  - All low-level control software has been revisited, upgraded.
    - Includes moving power supply control software from stand-alone code to an algorithm within PCS.
  - Physics algorithms required for commissioning have all been revised, updated, tested.
  - Now working on more advanced physics algorithms to support research.

# Operations-Side Activities to Support Operations:

## Boundary Physics Operations

- New boronization (dTMB) system will be used.
  - Designed, fabricated, installed...awaiting only final PLC software.
  - Plan for it to be available for initial research operations.
- LIThium EvaporatoR (LITER)
  - LITERs were carefully stored during the outage.
  - Mounting brackets have been reinstalled.
  - New Argon Dump system being designed for installation before LITER use.
    - Vessel has been prepped to accommodate that system.
- High-conductance divertor gas injection lines (2) have been installed
  - Support radiative divertor studies
- Gas delivery system upgrades.
  - Moving to a uniform system of valve drive technologies.
  - All valves to be commanded from PCS.
- Materials Analysis Particles Probe (MAPP) probe has been installed.
  - Allows material samples to be exposed to the plasma and then examined in-situ with surface science techniques.

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# **Projected Impact of the Aquapour/CTD-425 Composite Material**

# Background: We Have Plans to Increase the Field, Current & Pulse Duration Over 3 Years

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1<sup>st</sup> year goal: operating points with forces up to ½ the way between NSTX and NSTX-U, ½ the design-point heating of any PF/TF coil (~75% for OH)

2<sup>nd</sup> year goal: Full field and current, but still limiting the PF/TF coil heating

3<sup>rd</sup> year goal: Full capability

**1<sup>st</sup> and 2<sup>nd</sup> year goals not affected materially by composite material**

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**This scenario most likely to be affected**

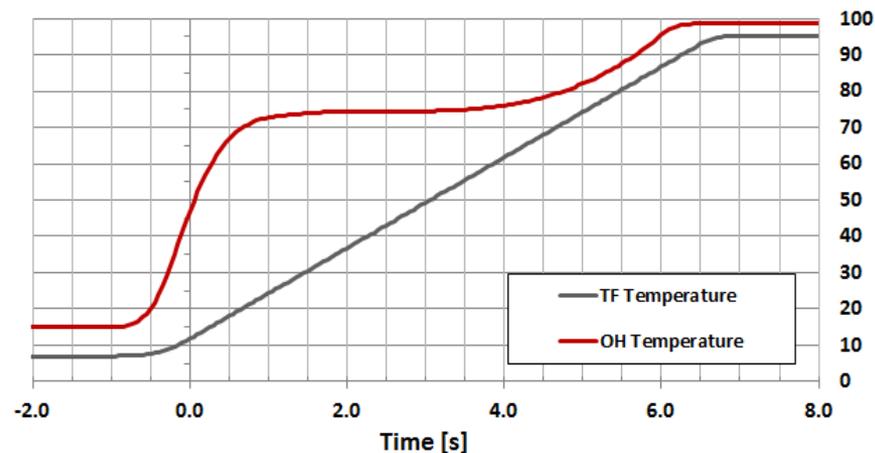
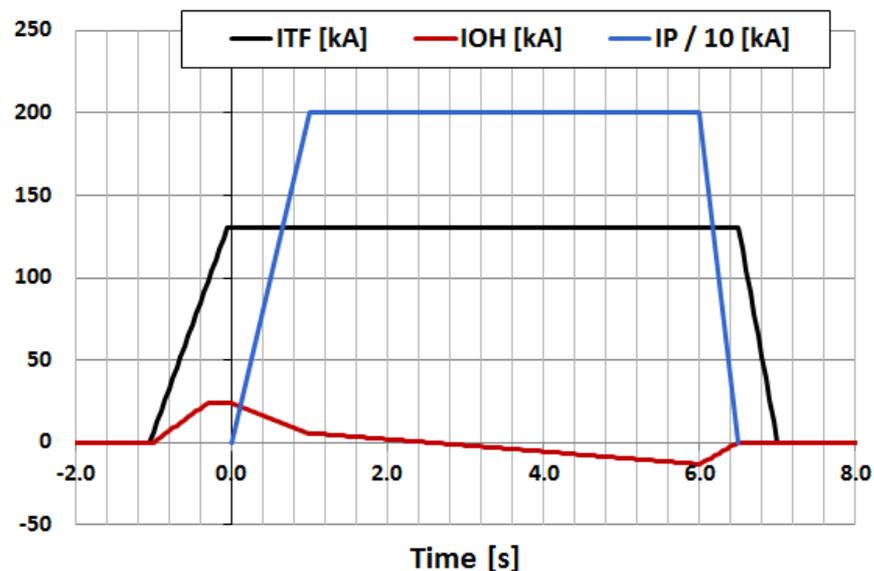
**1<sup>st</sup> and 2<sup>nd</sup> year goals not affected materially by composite material**

# Summary Statement:

## Physics Program Largely Unaffected By Requirement $T_{TF} < T_{OH}$

- Illustrative Example: 2 MA, 1T, 5 second.
- TF Coil:
  - Current is constant
  - Temperature is linear
- OH Coil:
  - current has a zero-crossing
  - Temperature has an “S-Shaped” curve.
- Options for maintaining  $T_{TF} < T_{OH}$ .
  - **Pre-heat the OH coil** using currents (or water) before the TF turns on.
  - Control the shape of the OH S-curve by **adjusting the amount of pre-charge**.
- In this example,
  - Full 24 kA pre-charge
  - Pre-charge duration is extended to provide heating.

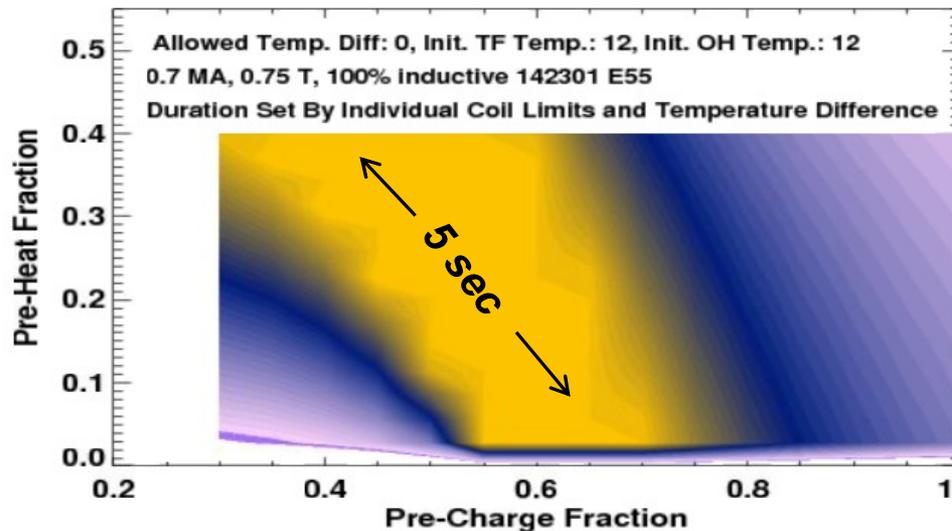
$$H_{98} = 1.2, f_{\text{Greenwald}} = 0.75, P_{\text{NBI}} = 8\text{MW}, \beta_N = 4.6$$



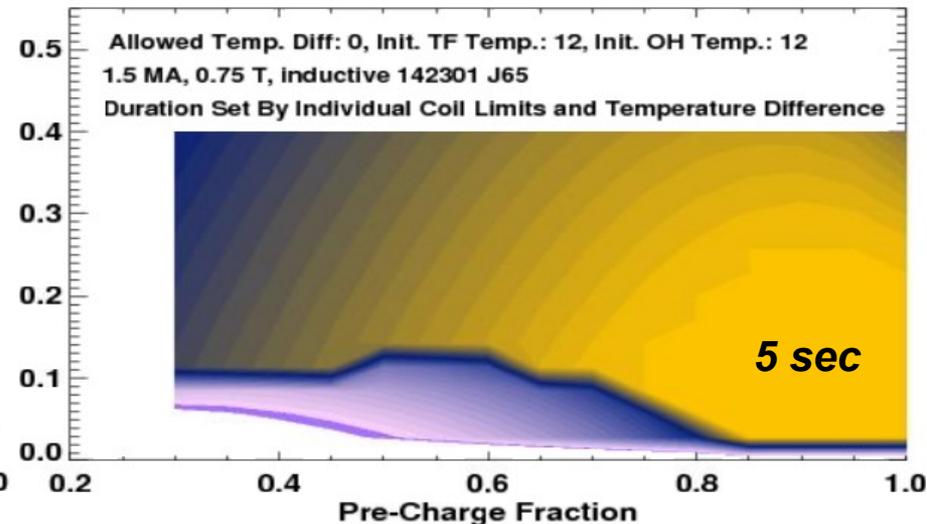
# Initial Operations Will Be Largely Unaffected By Aquapour/CTD-425 Composite

- Scan two variables in these studies:
  - Pre-heat level, quantified as the fraction of the full OH coil I<sup>2</sup>t limit used before the shot starts.
  - Pre-Charge fraction, quantified as the fraction of 24 kA used.

**0.7 MA, 0.75 T, 100% Non-Inductive**



**1.5 MA, 0.75 T, Partial Inductive**



- Resistive pre-heat of ~15% provides operating room for 0.75 T scenarios typical of the first year.
  - Same as starting the coil at ~23 C

# Limitations Become Apparent, but Manageable, for 2 MA Cases

**Reason: These Often Required the Full OH I<sup>2</sup>t**

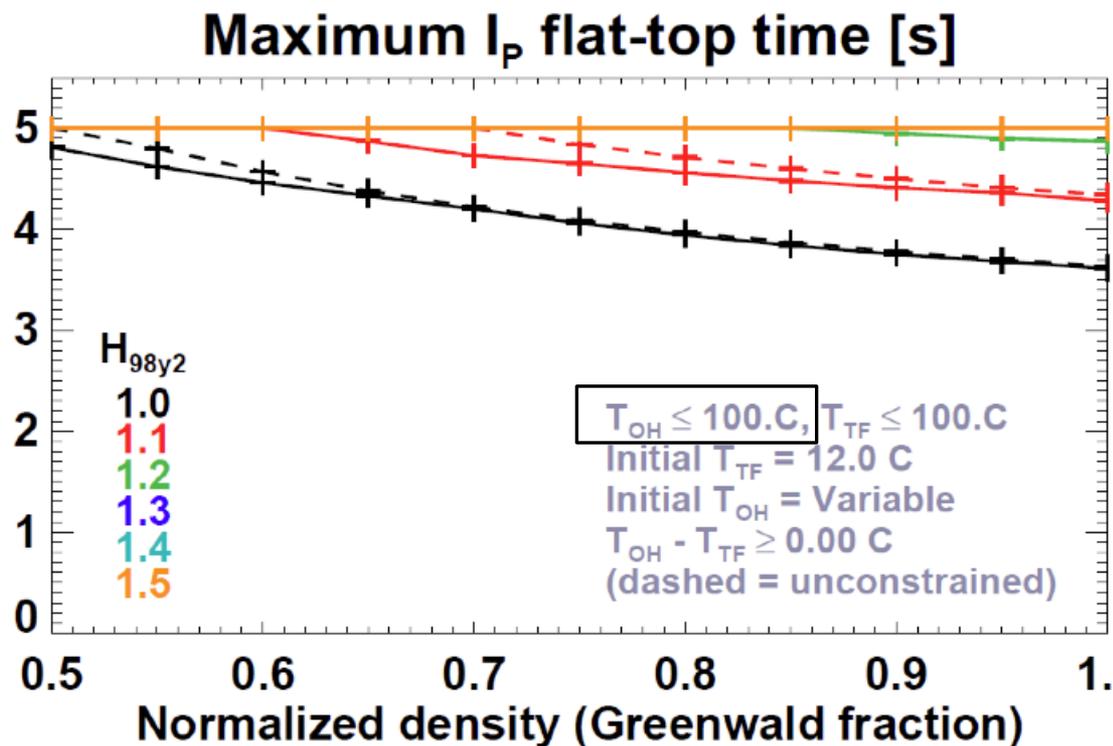
- 0D study as a function
  - Confinement ( $H_{98}$ )
  - Density (Greenwald)
- Determine via optimizer the optimum initial OH temperature, *limiting the maximum to 100 C.*
- Result:
  - Typically only a 0.2 second (or less) reduction in pulse length, provided the pre-heat is optimized
  - $H_{98} \sim 1.2$  needed for reliable 5 sec operation for any constraint on relative temperatures.

**Solid:**

$T_{TF} < T_{OH}$  applied as a constraint

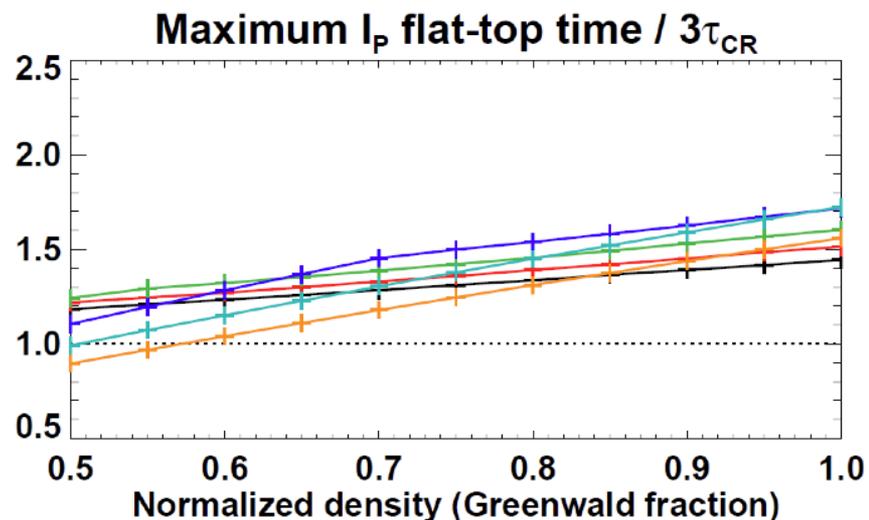
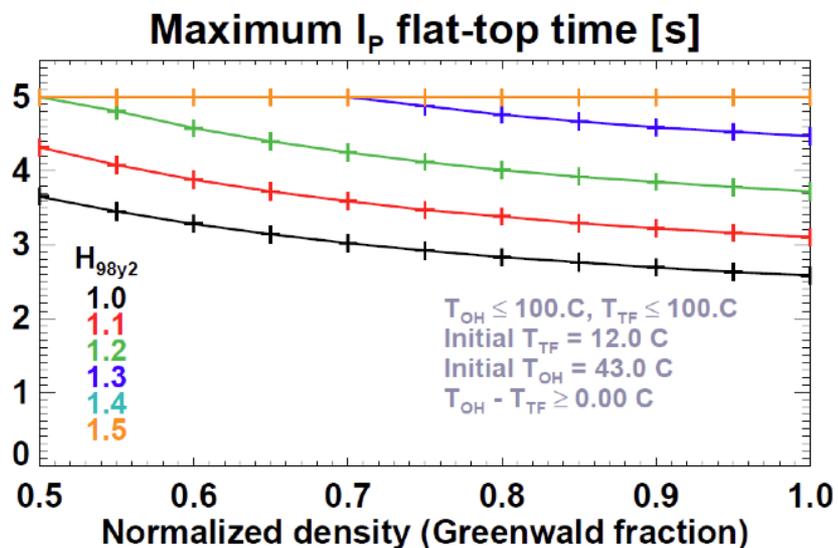
**Dashed:**

No constraint on relative temperatures



## Second Optimization: Find Initial OH Temperature that Guarantees Current Profile Equilibration for a range of 2 MA Configurations

- Two assumptions in this study:
  - The important normalization for the discharge duration is the  $3\tau_{CR}$ .
  - It will be an imposition to change the initial OH temperature all the time, so need to find an optimal single value.
- Fix the initial OH temperature to 43 C
  - Could be achieved, for instance, by a “standard” OH current pulse, or the water pre-heater.
- Durations in physical units lowered for  $H_{98y2}=1$ , but are *greater than  $3\tau_{CR}$*  for *essentially all densities and confinement*.
- 110 C operation on the OH largely eliminates the composite material as an issue.
  - Optimal initial temperature drops to 36 C.
  - Plasma thermal energy confinement and CD physics, not the composite material, would have the dominant impact on the pulse duration at 2 MA.
  - Under engineering evaluation, looking quite feasible.



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# More Scientific Organization

# NSTX-U Milestone Schedule for FY2016-18

(see updated Milestone web-page for additional detail / text)

	FY2016	FY2017	FY2018
Run Weeks:	Incremental 14 16	16 18	12 16
<b>Boundary Science + Particle Control</b>	<p>R16-1 Assess H-mode confinement, pedestal, SOL characteristics at higher <math>B_T</math>, <math>I_p</math>, <math>P_{NBI}</math></p>	<p>R17-1 Assess scaling, mitigation of steady-state, transient heat-fluxes w/ advanced divertor operation at high power density</p> <p>R17-2 Assess high-Z divertor PFC performance and impact on operating scenarios</p>	<p>R18-1 Assess impurity sources and edge and core impurity transport</p> <p>IR18-1 Investigation of power and momentum balance for high density and impurity fraction divertor operation</p>
<b>Core Science</b>	<p>R16-2 Assess effects of NBI injection on fast-ion <math>f(v)</math> and NBI-CD profile</p>	<p>R17-3 Assess <math>\tau_E</math> and local transport and turbulence at low <math>v^*</math> with full confinement and diagnostic capabilities</p>	<p>IR18-2 Assess role of fast-ion driven instabilities versus micro-turbulence in plasma thermal energy transport</p> <p>Begin ~1 year outage for major facility enhancement(s) sometime during FY2018</p>
<b>Integrated Scenarios</b>	<p>R16-3 Develop physics + operational tools for high-performance: <math>\kappa</math>, <math>\delta</math>, <math>\beta</math>, EF/RWM</p>	<p>IR17-1 Assess fast-wave SOL losses, core thermal and fast ion interactions at increased field and current</p> <p>R17-4 Develop high-non-inductive fraction NBI H-modes for sustainment and ramp-up</p>	<p>R18-2 Control of current and rotation profiles to improve global stability limits and extend high performance operation</p> <p>R18-3 Assess transient CHI current start-up potential in NSTX-U</p>
<b>FES 3 Facility Joint Research Target (JRT)</b>	<p>C-Mod leads JRT Assess disruption mitigation, initial tests of real-time warning, prediction</p>	<p>DIII-D leads JRT Understanding Detachment</p>	<p>NSTX-U leads JRT TBD</p>

# Topical Sub Groups (TSGs) play major role in governing the research program

- Led brainstorming, organization, writing of 5 year plan
- Address highest priority scientific issues through discussion and consensus at open meetings
- Organize the NSTX-U Research Forum sessions.
- Draft scientific milestones
- Propose and execute experiments
- Define facility and theory resources to achieve research goals
- Aid dissemination of results with Physics Analysis Division
  - Journal publications, invited talks, seminars, colloquia, conferences, ITPA, BPO
- Provide summaries of scientific progress at NSTX-U team meetings and other venues to promote discussion
- Assist and report to the NSTX-U Program and Project directors

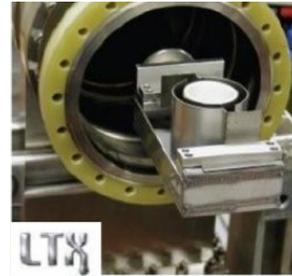
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# Long Term Upgrades

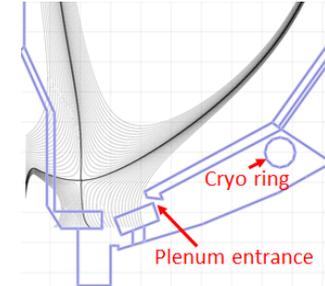
# NSTX-U facility enhancements proposed for 5 year plan support FESAC Tiers/Priorities

- Improved particle control tools
  - Control D inventory, rapidly trigger ELMs to expel impurities (*Transients, PMI*)
  - Low  $v^*$  to understand ST confinement to support FNSF, validation (*FNSF, Predictive*)
- Disruption avoidance, mitigation (*Transients, Predictive*)
  - Massive gas injection, detect halos, disruptions, control  $v_\phi$ , RWM, ELM
- ST start-up and ramp-up tools (*FNSF*)
  - ECH to raise start-up plasma  $T_e$  to enable FW + NBI + BS  $I_p$  ramp-up
  - Test EBW-CD start-up, sustainment
  - Start-up/ramp-up critical for ST-FNSF
- Begin transition to high-Z PFCs, assess flowing liquid metals (*PMI, FNSF*)
  - Plus divertor Thomson, spectroscopy

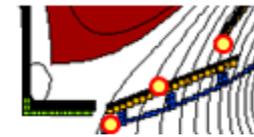
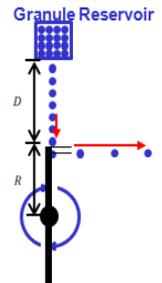
Upward Li evaporator



Divertor cryo-pump

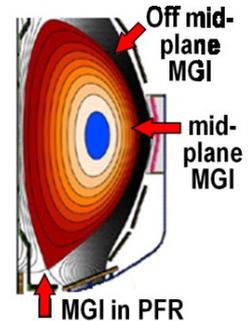
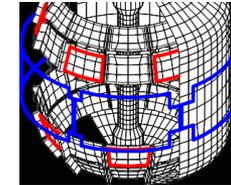


Li granule injector (LGI)



Extended low-f MHD sensor set

Midplane + off-midplane non-axisymmetric control coils (NCC)



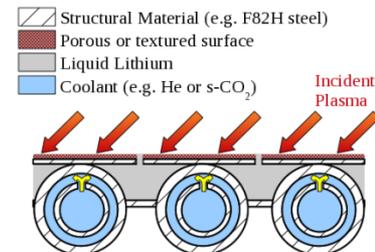
1MW 28 GHz gyrotron



High-Z tiles



Actively-supplied, capillary-restrained, gas-cooled LM-PFC



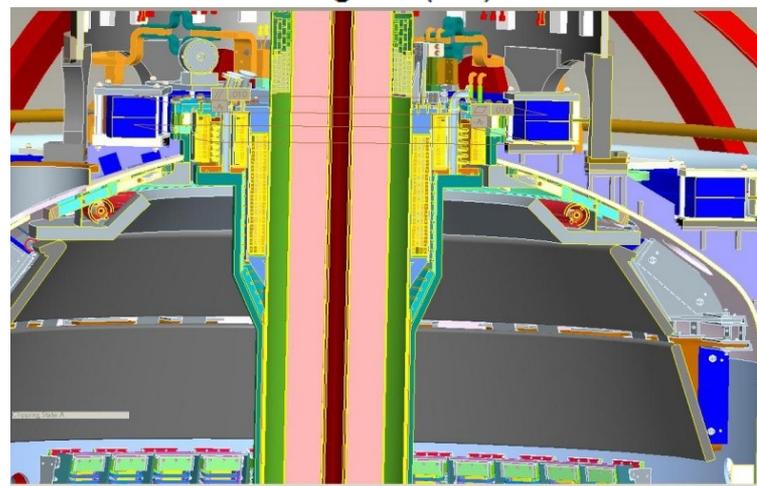
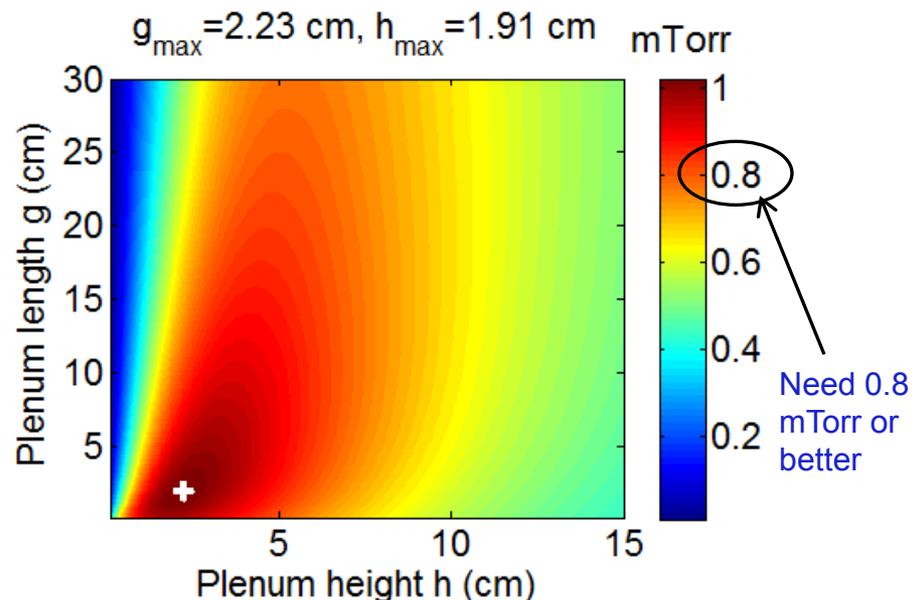
# 4 Major Upgrade Are Being Considered for the next 5 Years

- We need robust density control.
  - Proposed Upgrade: *Divertor cryopump*
- We need to develop high-performance ST scenarios compatible with reactor relevant plasma facing components (PFCs).
  - Proposed Upgrade: *High-Z PFC upgrade*
- We need to understand how a CHI formed plasma can be ramped to full current without solenoid induction.
  - Proposed Upgrade: *ECH system*
- We need to better understand and optimize error field correction, resonant magnetic perturbations for pedestal control, fast feedback for RWM control, rotation profile control.
  - Proposed Upgrade: *Additional 3D field coils*

For detailed justification, see: <http://nstx-u.pppl.gov/five-year-plan/five-year-plan-2014-18>

# Cryopump Physics Design to Provide Pumping over a Wide Range of Divertor Geometries and Core Densities

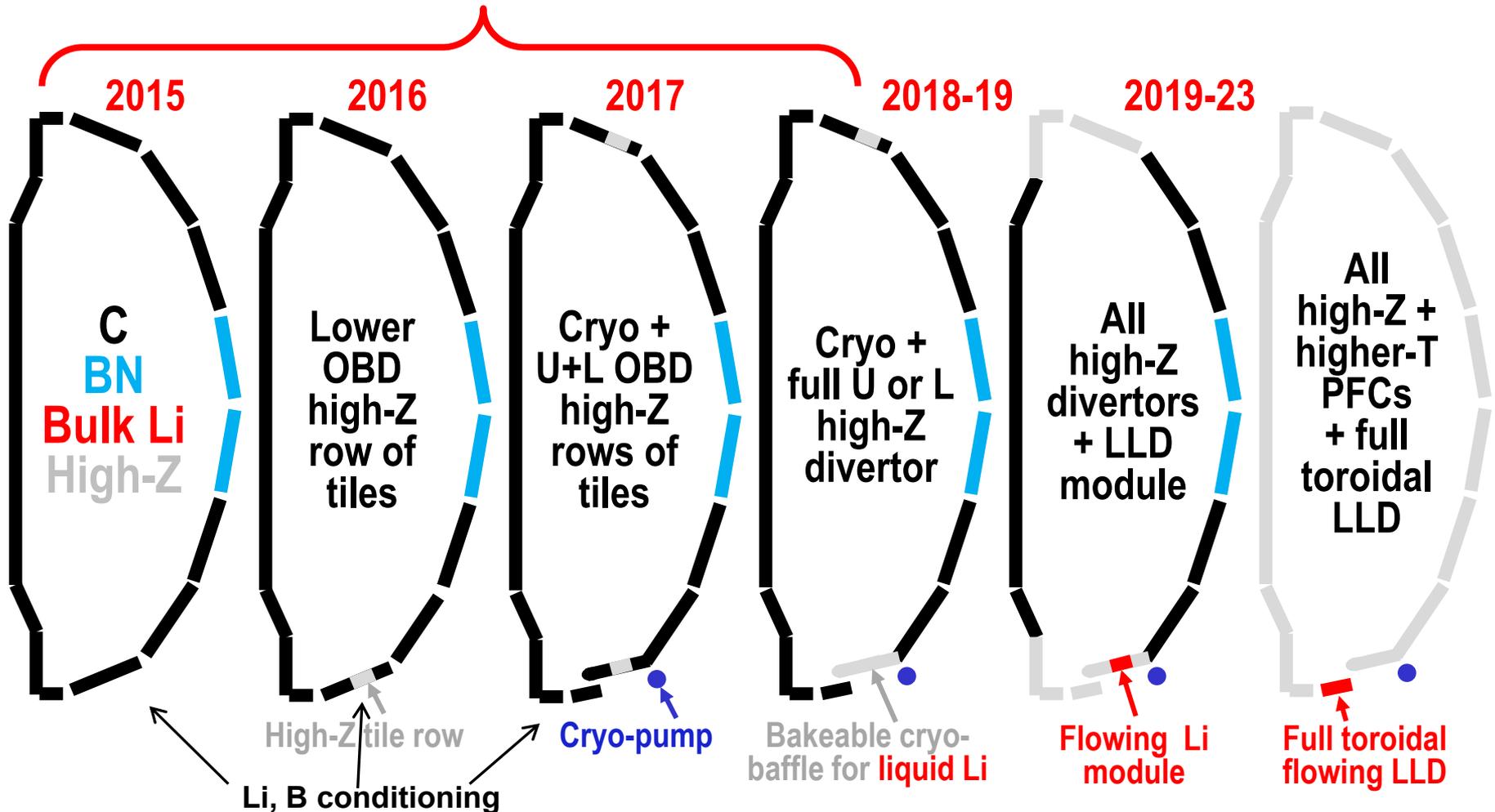
- Physics design completed in collaboration with ORNL<sup>1</sup>.
  - Defined the geometry, plenum sizes, ability to pump various geometries.
- Conceptual design process has been initiated:
  - Draft GRD has been formulated.
  - Initial designer sketches of in-vessel implementation completed.
  - Potential refrigerator systems and associated elements identified.
  - Goal is to to have the system available for the 2017 run campaign.



[1] [http://nstx.pppl.gov/DragNDrop/Scientific\\_Conferences/APS/APS-DPP\\_12/Contributed\\_Posters/PP8.00030\\_Canik\\_APS2012.pdf](http://nstx.pppl.gov/DragNDrop/Scientific_Conferences/APS/APS-DPP_12/Contributed_Posters/PP8.00030_Canik_APS2012.pdf)

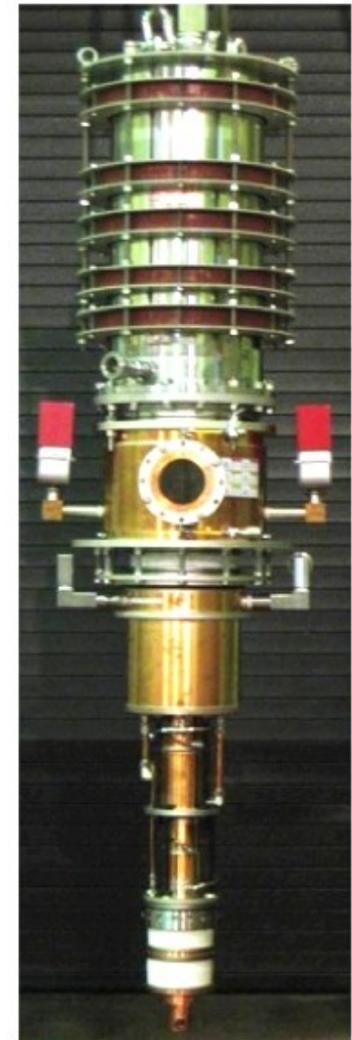
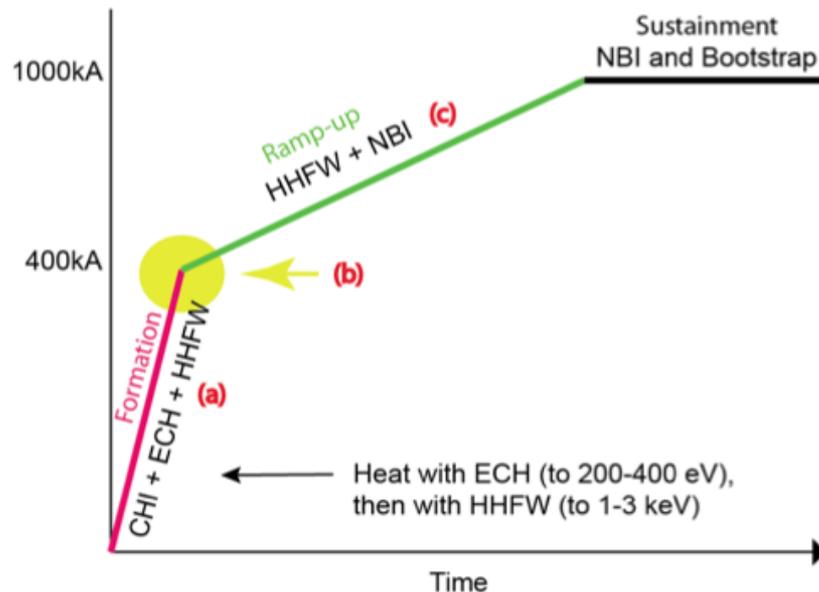
# Stages Plan Has Been Developed to Implement High-Z PFCs & Flowing Liquid Metal Systems

*Nominal 2014-18 5 year plan steps for implementation of cryo-pump + high-Z PFCs + LLD*



# 28 GHz Gyrotron System Will Facilitate Non-Inductive Ramp-Up

## NSTX-U Start-up and Ramp-up strategy



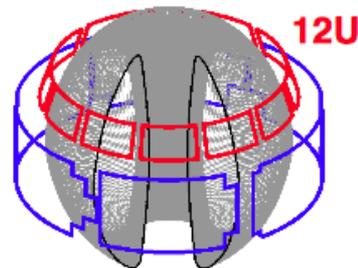
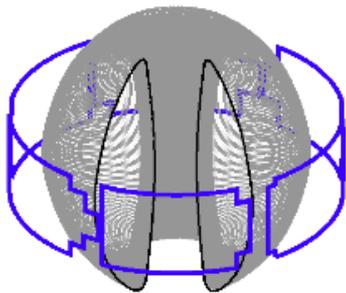
- Coaxial Helicity Injection can form a 200-400 kA seed plasma, but it is too cold and quickly decays.
- Use of ECH can “bridge the gap”, to where HHFW and then NB current drive can support the ramp and sustain the current.
  - 20-40% first pass absorption predicted by GENRAY.
- Goal of first ECH power in 2017 run

1 MW Tube Developed for Gamma 10

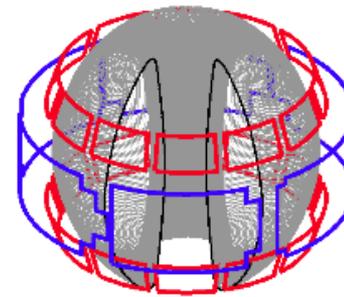
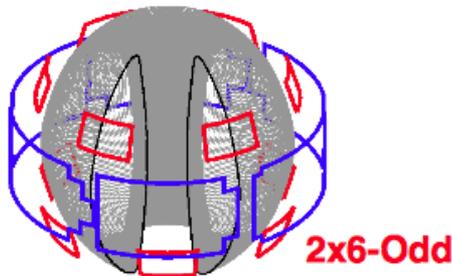
# 3D Coil Physics Design Targets a Range of Physics Objectives

- NCC = Non-axisymmetric Control Coil
- Evaluated three upgrade options based on numerous physics criterion
  - Magnetic breaking, error field control, fast RWM control, RMP applications.
- Initial findings shows that while the 2x12 options is best, a phase implementation starting with the 2-6-Odd approach may be a good intermediate step.

**Existing  
Midplane coils**



**NCC Options**



**2x12**

*Targeting the Out  
Years of the 5 Year Period*

# Collaborators Play a Key Role in the NSTX-U Research Program

- University, national lab, and business collaborations for both data analysis and facility upgrades (diagnostics, gas injectors,...).
- Collaborations reviewed & renewed on a 3-4 year cycle.
- Key documents:
  - Record of Discussion: documents communications between PPPL and collaborator during the formulation of DOE proposal, including estimates of PPPL resources to support collaboration is funded
  - Record of Agreement: agreed commitments of resources, equipment, and facilities by collaborator and PPPL.
  - Data Usage Agreement: access to and publication of data.
- PPPL generally provides the vacuum interface, floor space, AC power & other services for diagnostics.
  - Collaborator provides the diagnostic itself, typically including data acquisition.
- Collaborators have the same safety & training requirements as PPPL employees.
  - And their systems have the same design reviews and work package requirements.

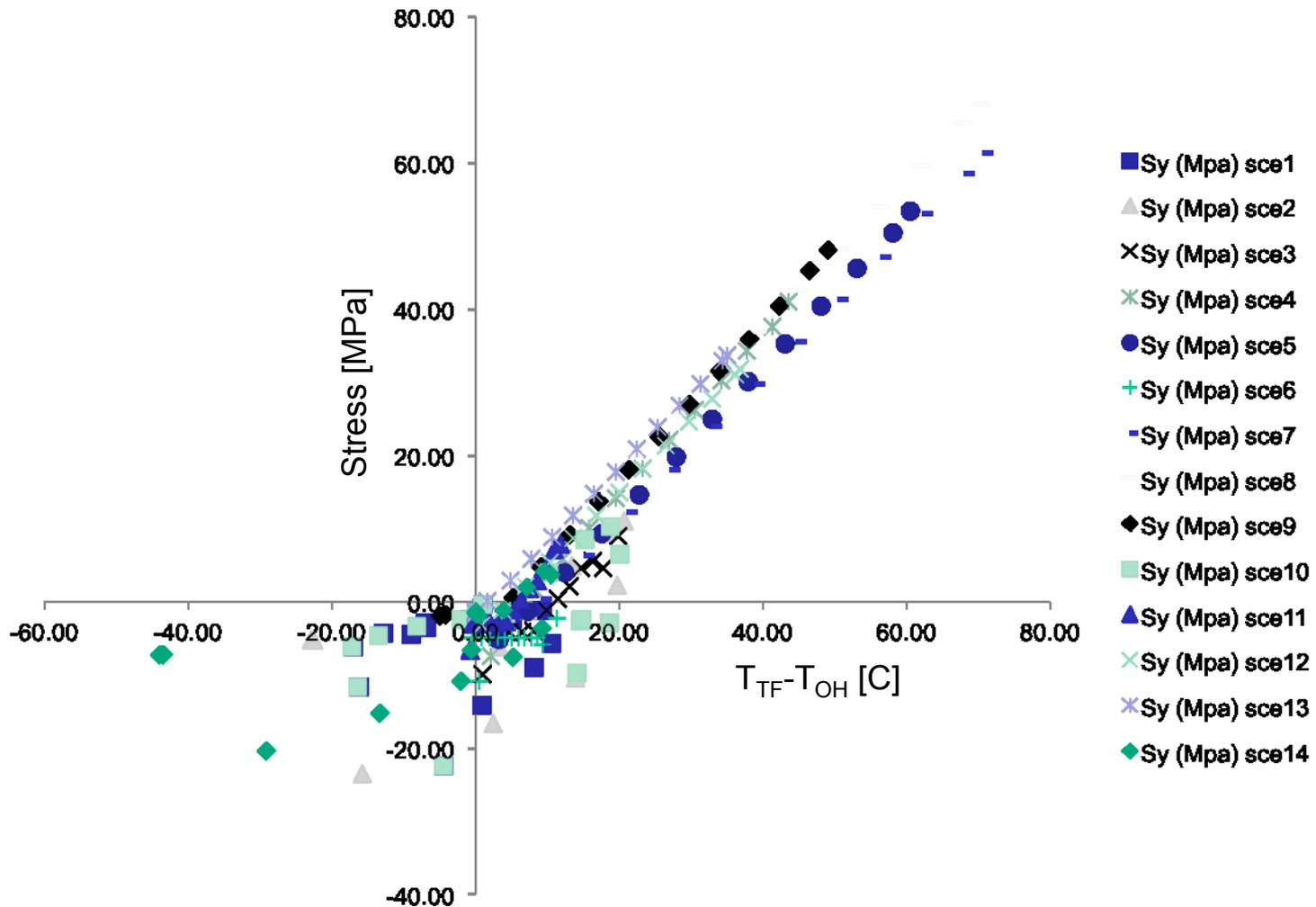
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# Analysis Justification for $T_{TF} < T_{OH}$ Rule

# Analysis Supports the Use of Temperature Differentials for The Initial Protection Scheme: Method

- Created 14 different discharge scenarios.
  - Mostly 2 MA, 1T, but a few at lower field and current.
  - Many variations in the pre-charge and pre-heat.
  - All had the TF temperature eventually exceed the OH temperature, sometimes by a large amount.
    - So are useful for defining protection scheme.
  - Had a wide range of OH states during the time when  $T_{TF}$  exceeded  $T_{OH}$  by 0-10 C.
- Used ANSYS to analyze the OH stress at 18 times in each of the discharge scenarios.
  - 14x18=252 combinations of stress, temperature difference, OH state
- Motivation: Find a bounding curve for the OH stress that is a function of only the temperature difference.

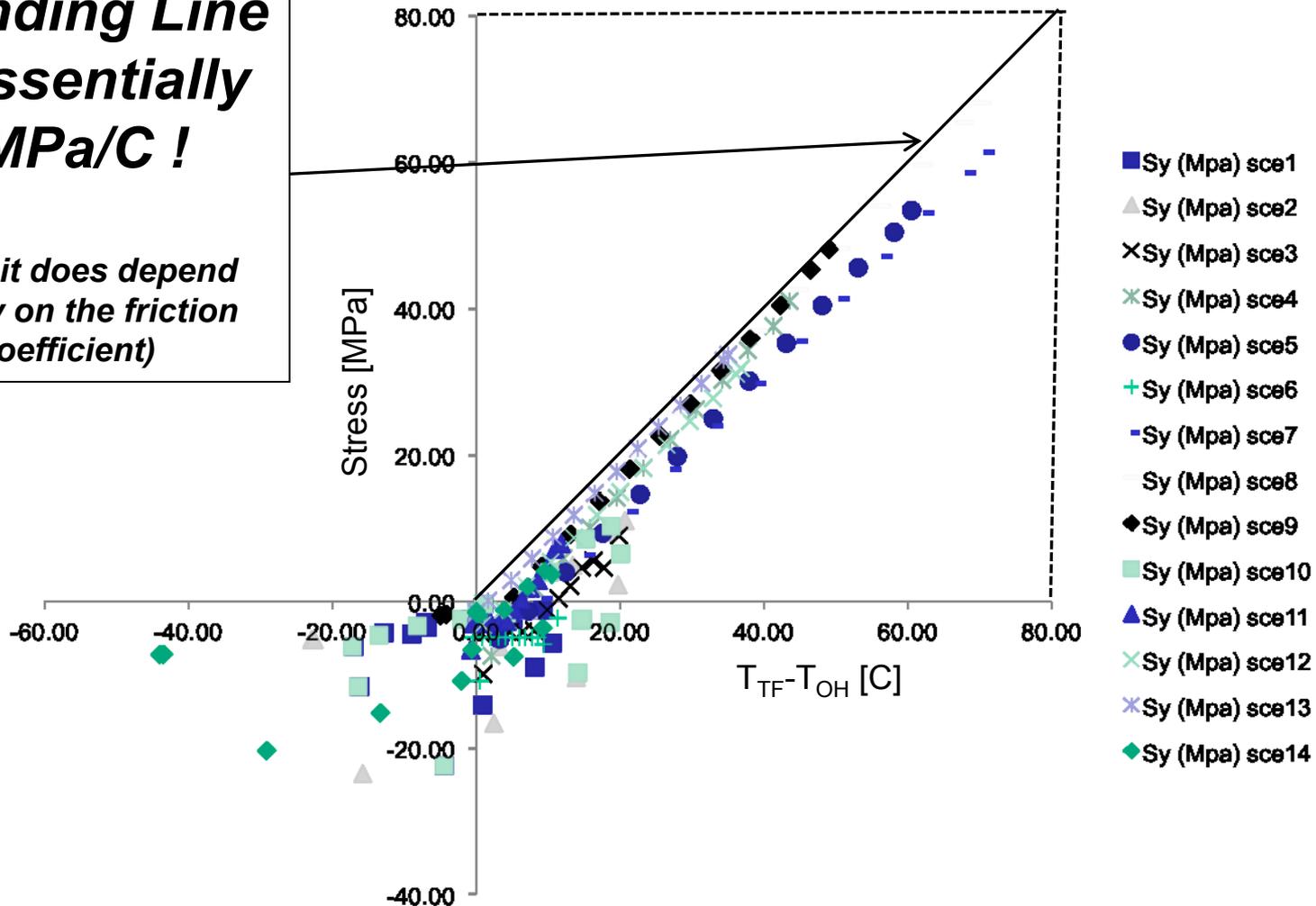
# Analysis Supports the Use of Temperature Differentials for The Initial Protection Scheme: Result (I)



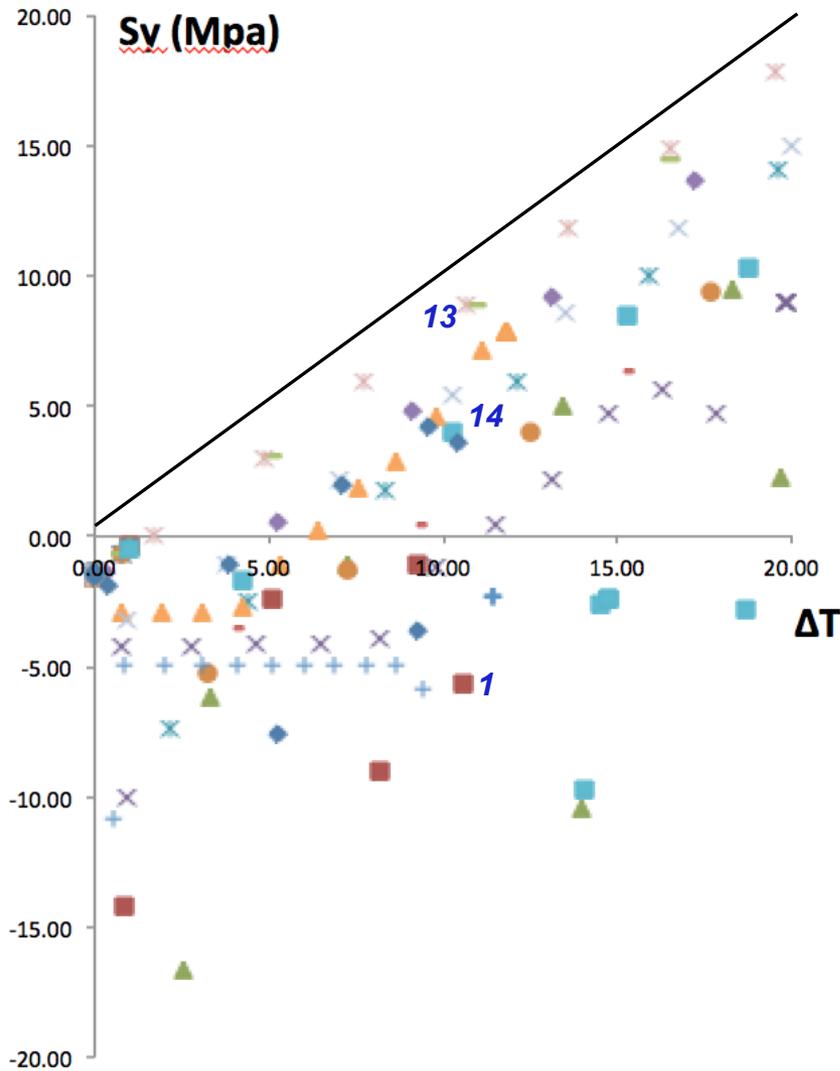
# Analysis Supports the Use of Temperature Differentials for The Initial Protection Scheme: Result (I)

**Bounding Line  
is Essentially  
1MPa/C !**

*(Note, it does depend  
weakly on the friction  
coefficient)*



# Analysis Supports the Use of Temperature Differentials for The Initial Protection Scheme: Result (II)



- Unity-slope bounding line holds over the temperature different region of interest ( $0 < \delta T < \sim 20$ ).
- Large variation under that line, due to:
  - The OH state
  - Path dependence.

# DCPS Will Be Used to Enforce This Temperature Difference

- Operating engineer & water-systems PLC enforce that the coils be cooled to a pre-defined set-point at the start of the discharge.
- Coil temperature evolution computed in DCPS based on current measurements.
  - Compute the temperature difference at the  $i^{\text{th}}$  step:  $\delta T_i = T_{\text{TF},i} - T_{\text{OH},i}$
- Consider the heating that would occur in the event of a fault:
  - $\delta T_{\text{TF},\text{fault},i} = I_{\text{TF},i}^2 C_{\text{TF}}$ ,  $\delta T_{\text{OH},\text{fault},i} = I_{\text{OH},i}^2 C_{\text{OH}}$  (OH may or may not heat up more than the TF)
  - $\delta T_{i,\text{fault}} = (T_{\text{TF},i} + \delta T_{\text{TF},\text{fault},i}) - (T_{\text{OH},i} + \delta T_{\text{OH},\text{fault},i})$
- At each cycle, compare both  $\delta T_i$  and  $\delta T_{i,\text{fault}}$  to the defined limit (0 in the first year).
- Algorithm accounts for both instantaneous heating, and fault heating, while only relying on coil current measurements.
- Temperature evolution algorithm will be calibrated against outlet water temperature and potentially other measurements.

# What Happens if the OH is Allowed to Operate up to 110 C?

## 1: Kinder Operating Window

### Both Simulations

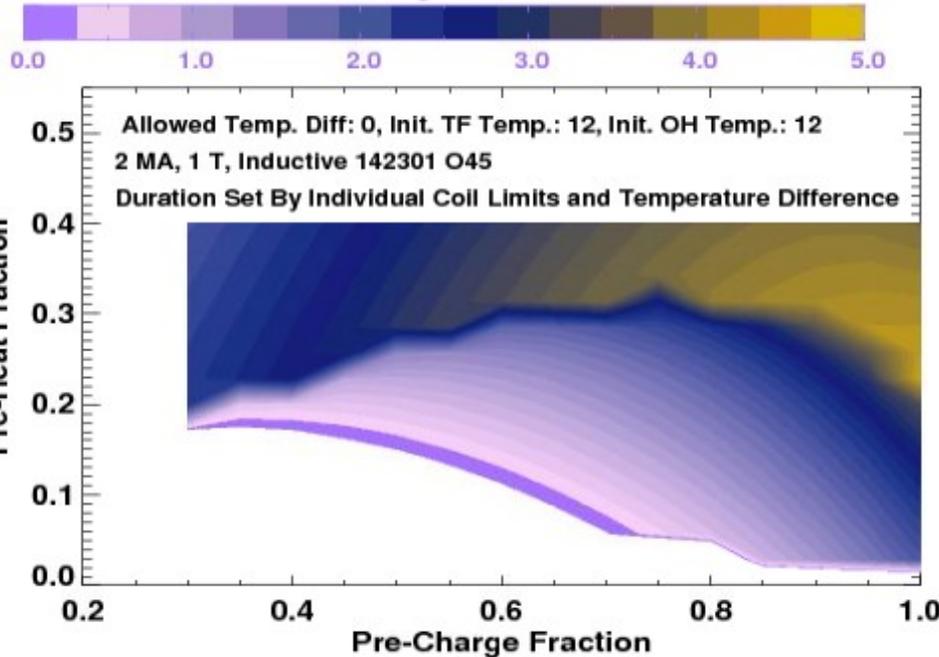
Initial Coil Temperatures of 12 C, with Resistive Pre-Heating Method

$T_{TF} < T_{OH}$  maintained

2 MA, 1T, H~1.05, allows 5 sec. shot *without* relative temperature constraint

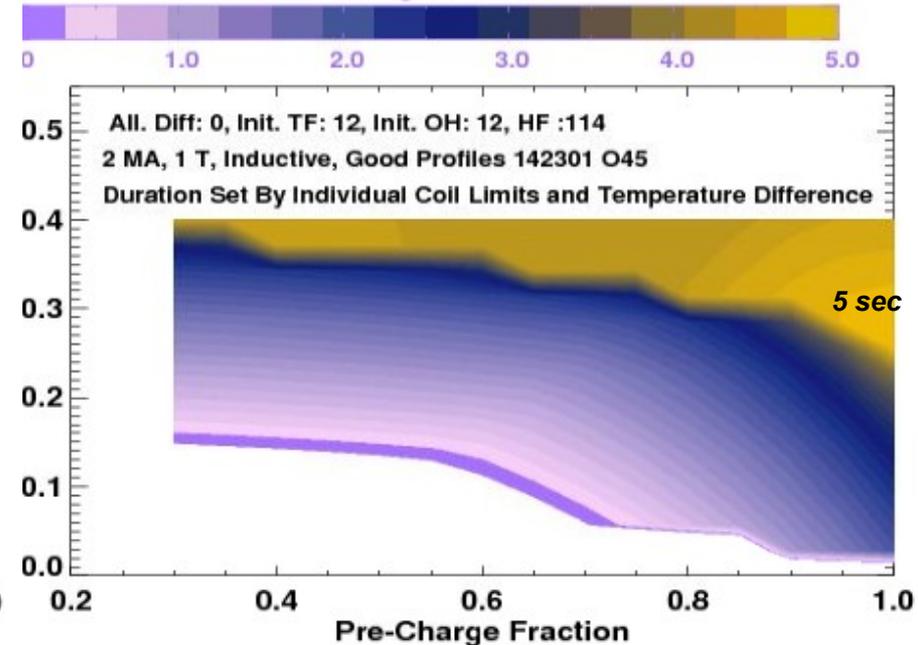
$T_{OH}$  limited to 98 C

Discharge Duration



$T_{OH}$  limited to 110 C

Discharge Duration



**For this confinement multiplier, 5 sec. operation restored with with 110 C max. OH temperature**

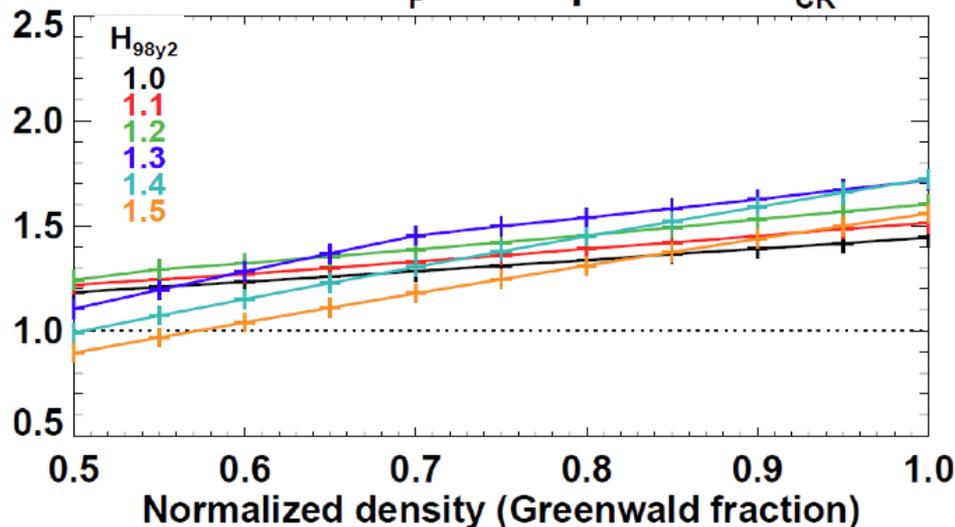
# What Happens if the OH is Allowed to Operate up to 110 C?

## 2: More robust access to $t_{\text{discharge}} > 3\tau_{\text{CR}}$

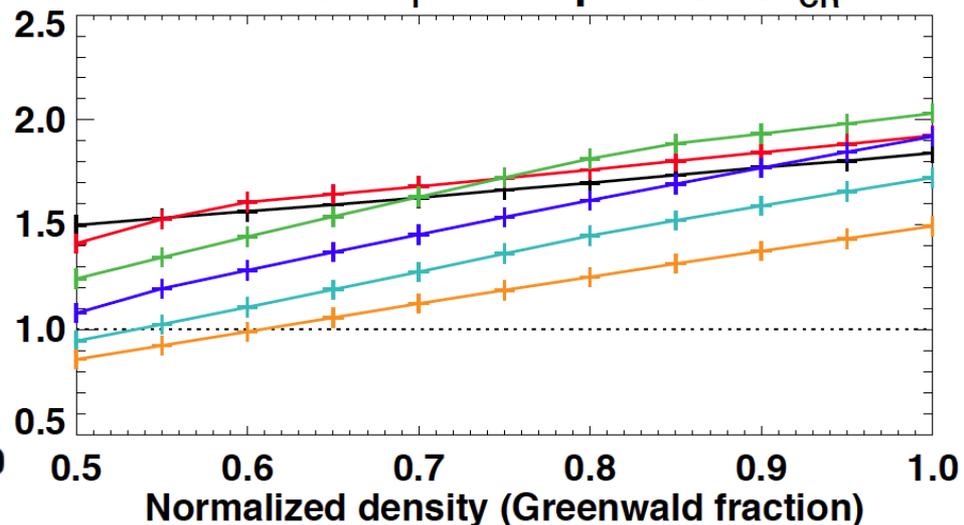
$T_{\text{OH}} < 100$  C,  $T_{\text{TF}} < 100$  C  
Initial TF temperature: 12 C  
OH Pre-heat to 43 C

$T_{\text{OH}} < 110$  C,  $T_{\text{TF}} < 100$  C  
Initial TF temperature: 7 C  
OH Pre-heat to 35 C

Maximum  $I_p$  flat-top time /  $3\tau_{\text{CR}}$



Maximum  $I_p$  flat-top time /  $3\tau_{\text{CR}}$

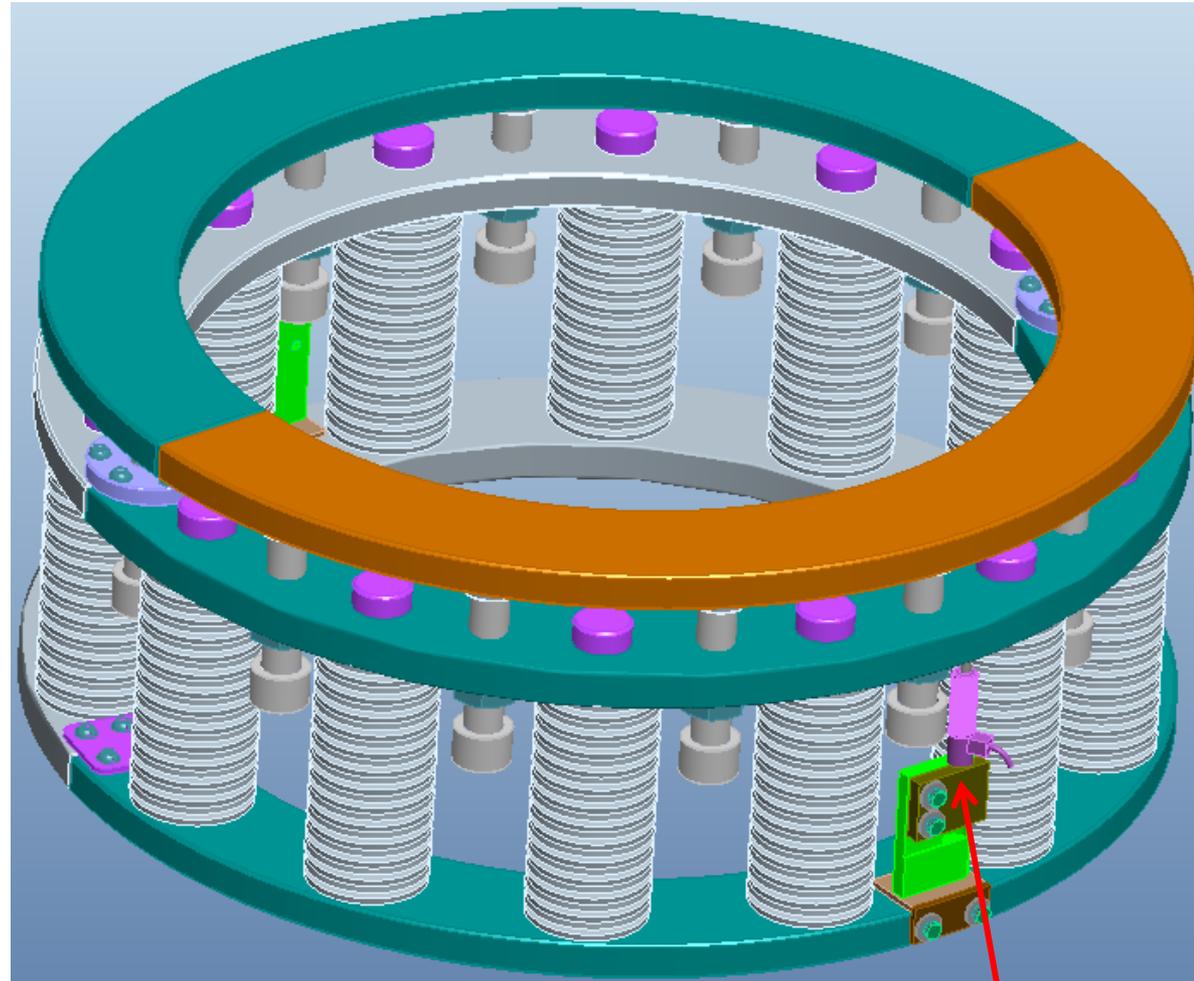
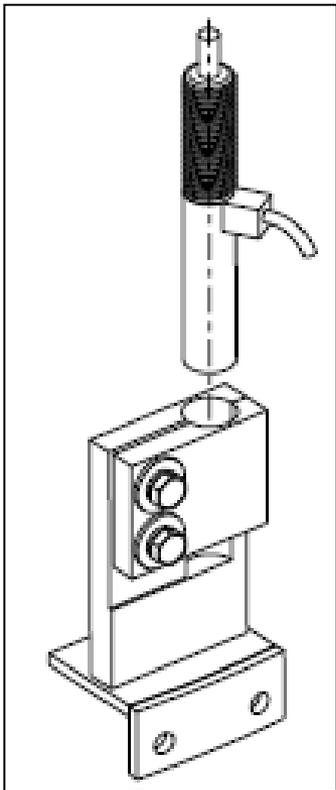


In this configuration, only 0.2-0.4 sec of absolute discharge duration lost, despite the **fixed initial OH temperature**

# OH Solenoid Thermal Growth Sensors Implemented

## FOD sensors will monitor OH solenoid growth

- Originally motivated by desire to monitor the pre-load.
- Two fiber optic displacement (FOD) sensors to be installed at 180° apart.
- The fixtures can be installed now and the sensors will be installed after the center stack is installed.

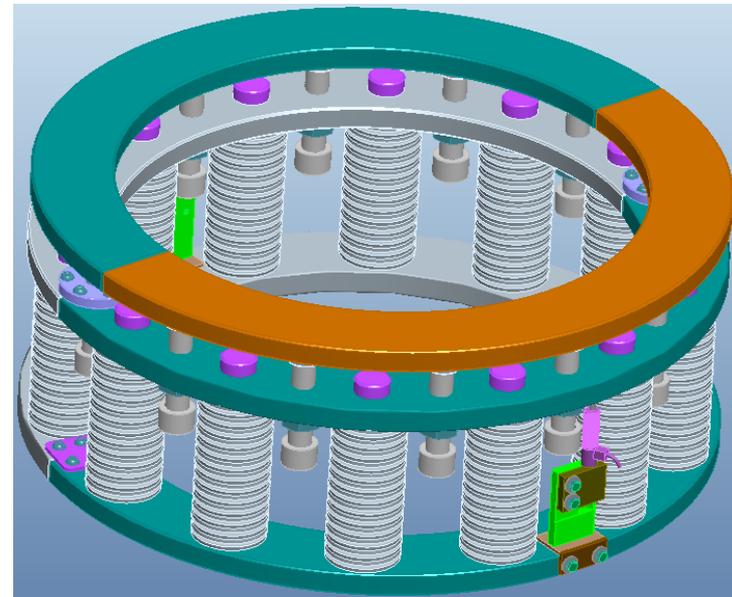
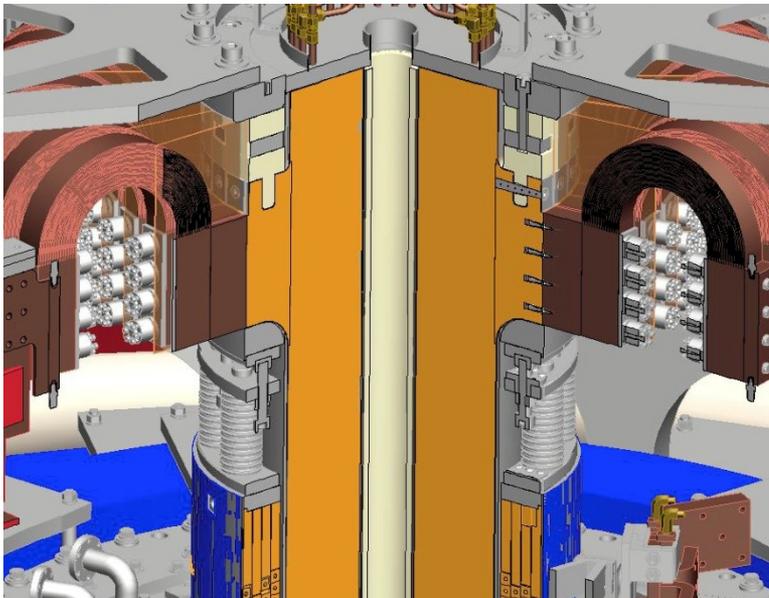


***Intent these to be used for trending data and analysis verification, not realtime protection***

**FOD Sensor**

# Permanent Aquapour/CTD-425 Composite Does Have Some Advantages

- OH coil will stay well centered on the TF bundle.
  - Eliminates the need for centering shims.
- OH pre-load mechanism is more robust.
  - OH pre-load provided by Belleville washer stack pushing on the TF coil flags.
  - 20 klb limit on the OH  $F_z$  determined by the hot-TF, cold-OH case.
  - By eliminating this case, the  $F_z$  limit is increased to 30 klb.
    - Provides additional headroom for control oscillations.



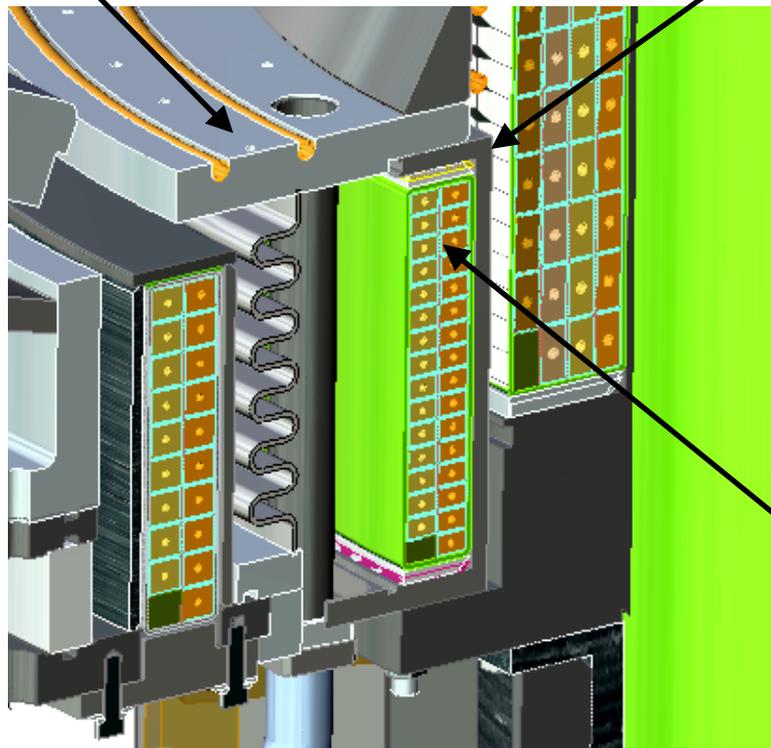
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# Bakeout & PF-1b

# Temperature Distribution Near PF-1b and the Horizontal Target Must be Managed

We want it hot here...350 C if possible  
(Where the tiles are)

We want to avoid thermal gradients here  
(Where there is a small stitch weld holding the casing to the PF-1b mandrel)

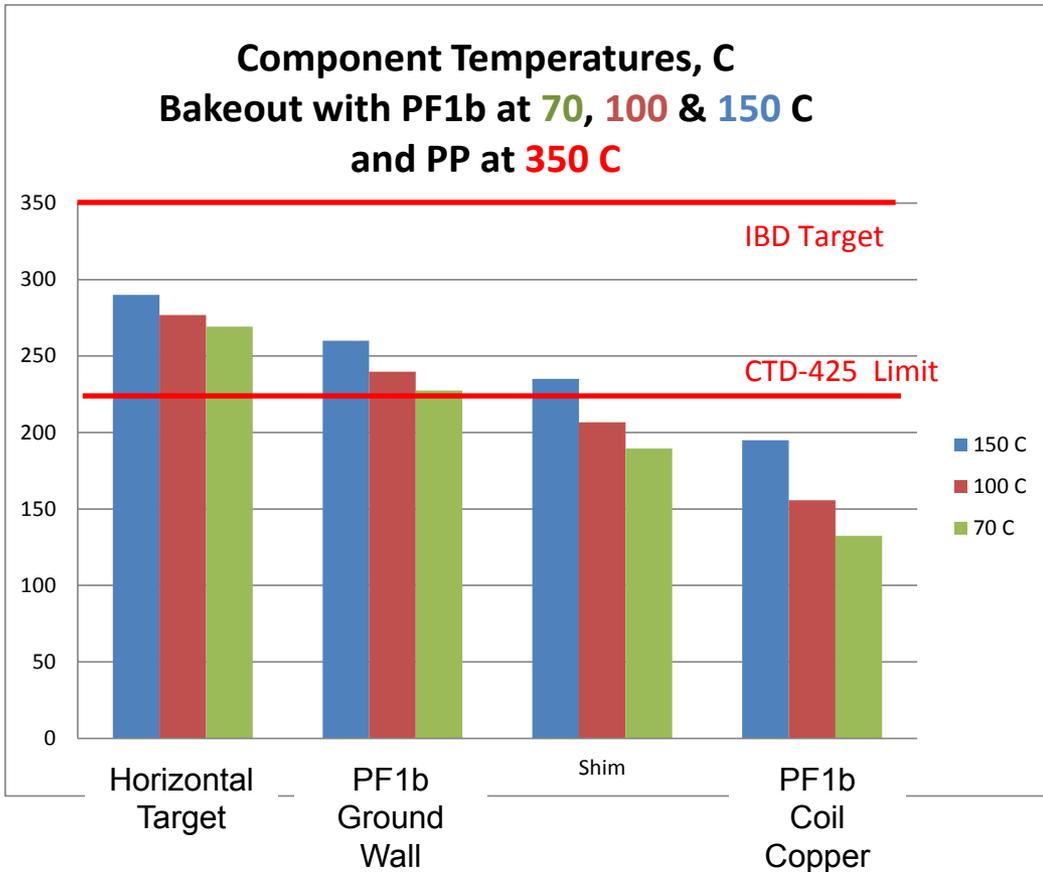


We want it cool here  
(On the coil turn-and-ground-insulation)

# Flowing 70 C Water Through PF-1b Appears to be the Optimal Compromise

- Options for control are limited:
  - Flowing hot helium on copper cooling tubes was infeasible.
- Optimal solution appears to be 70 C water in PF-1b
  - Keeps insulation below 225 limit.
  - Limits weld stress
  - Allows the horizontal target to reach ~270 C.
- Heat exchanger solution being implemented to provide correct water to the PF-1b during next bakeout.

Temperature at Four Locations as a Function of the PF-1b Water Temperature

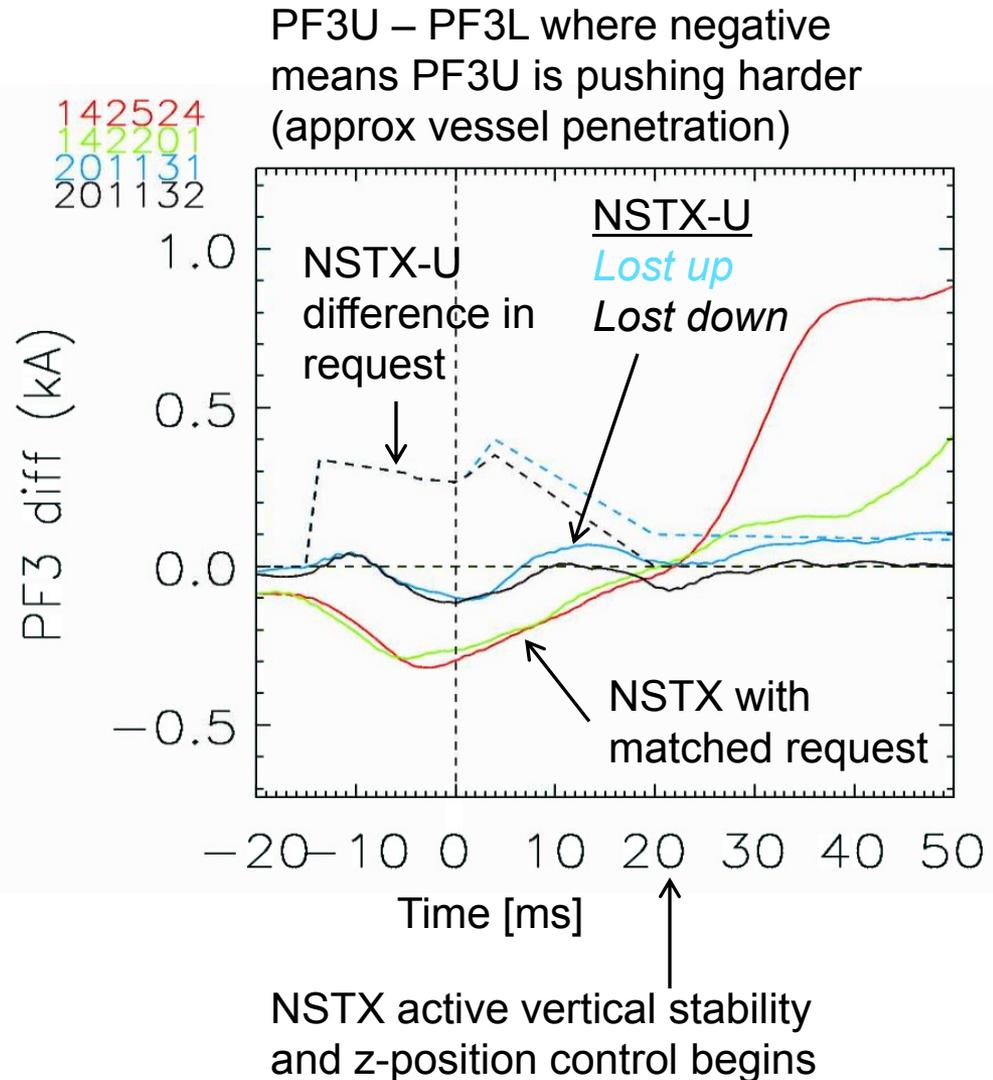


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# More on Plasma KPP

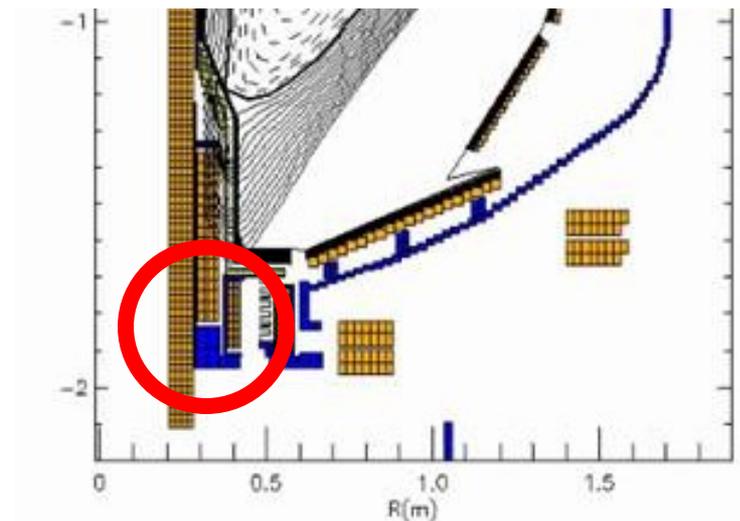
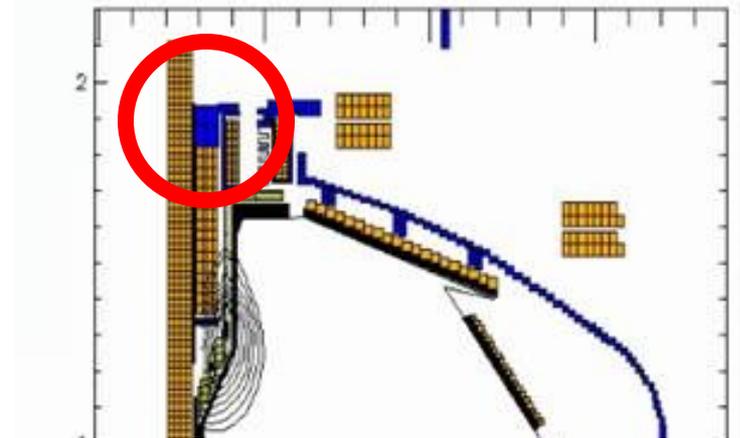
# When we Request the Same Current in PF3U and PF3L, the Plasma Stays Below the Midplane – why?

- Factor #1: PF3U/L current control is not identical
  - Evidence that same imbalance existed in NSTX, but intrinsic vertical stability was still ok
- Factor #2: Differences between the upper and lower divertors
  - Reconstructions confirm imbalance in eddy currents



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- Factor #2: Differences between the upper and lower divertors
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# Startup Scenario Will Benefit from Reduced $V_{loop}$ When Operations Resume after Bakeout (and later Boronization(s))

- Induced currents in divertor challenge passive vertical stability
  - $V_{loop} \sim 4.5V$  is challenging
- It is expected that these plasmas were “dirty” due to unbaked tiles
  - CHERs sees bright C and O lines, some residual Li
  - No bright N lines
    - Good vacuum quality
- After bake, cleaner plasma will reduce  $V_{loop}$  requirements and improve intrinsic stability (lower li)
  - Energizing the PF1CU/L to cancel induced fields is an option if needed

